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GRAPHICAL AND TABULAR SUMMARIES OF DECAY CHARACTERISTICS FOR ONCE-THROUGH PWR, LMFBR, AND FFTF FUEL CYCLE MATERIALS

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#### GLOSSARY

FFTF Fast Flux Test Facility (located in Richland, Washington)

HLW High-level waste

LMFBR Liquid-metal fast-breeder reactor

LWR Light-water reactor

MTIHM Metric tons of initial heavy metal

ORIGEN2 A computer code for calculating the radionuclide composition and characteristics (radioactivity, thermal power, etc.) of nuclear materials such as spent fuel and wastes

ORMANG A computer program that processes ORIGEN2 output to produce publication-quality graphs and tables

PWR Pressurized-water reactor

SMW Fuel assembly structural material (cladding) waste

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### GRAPHICAL AND TABULAR SUMMARIES OF DECAY CHARACTERISTICS FOR ONCE-THROUGH PWR, LMFBR, AND FFTF FUEL CYCLE MATERIALS

A. G. Croff, M. S. Liberman, and G. W. Morrison

#### ABSTRACT

Based on the results of ORIGEN2 and a newly developed code called ORMANG, graphical and summary tabular characteristics of spent fuel, high-level waste, and fuel assembly structural material (cladding) waste are presented for a generic pressurized-water reactor (PWR), a liquid-metal fast breeder reactor (LMFBR), and the Fast Flux Test Facility (FFTF). The characteristics include radioactivity, thermal power, and toxicity (water dilution volume). Given are graphs and summary tables containing characteristic totals and the principal nuclide contributors as well as graphs comparing the three reactors for a single material and the three materials for a single reactor.

#### 1. INTRODUCTION

The purpose of this report is to present information concerning the decay characteristics of nuclear materials from three generic types of nuclear reactors. The decay characteristics considered are radioactivity (Ci), thermal power (W), and ingestion toxicity (m<sup>3</sup> of water required to dilute the material to standards specified in 10 CFR 20). The nuclear reactors considered are a pressurized-water reactor (PWR), a liquid-metal fast breeder reactor (LMFBR), and the Fast-Flux Test Facility (FFTF). All are assumed to be operated on a once-through fuel cycle; i.e., the FWR is fueled with enriched uranium, and the plutonium in the LMFBR and FFTF has not been previously recycled. All of the decay characteristics presented were calculated with the ORIGEN2<sup>1</sup> computer code.

This report represents in a variety of ways an extension of previous work<sup>2</sup> concerning decay characteristics. First, this document considers the characteristics of FFTF materials not available for the earlier report. Additionally, the toxicity of the nuclear materials, not previously reported, is included here. This report also includes, wherever appropriate, a comparison of the various materials and reactors. However, the greatest extension represented by this work is that the information is presented in graphical as well as tabular form. The graphical presentation of the information was greatly facilitated by the development of a computer program (ORMANG) that accesses the output of the ORIGEN2 computer code, locates the desired information, summarizes it, and produces graphs and listings, as seen in this report.

Section 2 provides a summary description of the ORIGEN2 reactor models that form the basis of the calculations and describes the portions of the spent fuel that are contained in the high-level waste (HLW) and the fuel assembly structural material (cladding) waste (SMW). The final section gives a brief description of the results and contains tables for locating a particular graph.

#### 2. DESCRIPTION OF REACTOR MODELS AND NUCLEAR MATERIALS

The purpose of this section is twofold: (1) to describe the generic reactor models that form the basis of the information provided in this report and (2) to give a physical description of the nuclear materials associated with each reactor.

### 2.1 Description of Generic Reactor Models

The three generic reactor models used in this report have all been documented in detail in earlier reports.<sup>3,4</sup> As a result, only a summary description of the reactors will be given. The summary quantitative data concerning the three reactors are given in Table 1.

The generic PWR used is based on the Westinghouse design,<sup>5</sup> with a fuel assembly containing a 17 x 17 array of fuel pins. It is fueled with  $UO_2$  in which the uranium has been enriched to 3.2 wt %. One-third of the core is assumed to be replaced during each annual refueling.

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,			LMPBI	R fuel reg	ion(s) <sup>a</sup>		FFTF fuel region(s)				
Parameter	PWR	со	AB	RB	CO + AB	CO + AB + RB	Inner core	Outer core	Total		
Electric power, MW(e)	1250	1145	34	71	1179	1250	0	0	0		
Thermal power, MW(t)	3800	3624	107	225	3731	3956	139	223	362		
Average specific power, <sup>b</sup> MW(t)/MTIHM	37.5	123.25	6.89	4.15	83.09	39.86	150.0	150.0	150.0		
Average fuel burnup, MWd/MTIHM	33,000	101,289	5660	7250	68,280	45,267	45,000	45,000	45,000		
Irradiation duration, full-power days	880	821.8	821.8	1643.6	821.8		300.0	300.0	300.0		
Refueling cycle length, full-power days	293.3	273.9	273.9	273.9	273.9	273.9	100.0	100.0	100.0		
Charge, kg/refueling cycle											
235 <sub>U</sub>	1081	16.4	10.3	18.1	26.7	44.8	5.1	7.7	12.8		
Total uranium	33,778	7 <b>99</b> 0	5166	905 <b>9</b>	13,156	22,215	717	107 <b>9</b>	1796		
Fissile plutonium <sup>C</sup>	0	1236	0	0.	1236	1236	183	362	545		
Total plutonium	0	1810	0	0	1810	1810	208	411	619		
Total (Th + U + Pu)	33,778	<b>98</b> 00	5166	9059	14,966	24,025	925	1490	2415		
Discharge, kg/refueling cycle											
235U	267	6.7	7.5	12.6	14.2	26.8			10.3		
Total uranium	32,173	7069	<b>49</b> 70	8662	12,039	20,701			1730		
Fissile plutonium <sup>C</sup>	210	10 <b>94</b>	159	315	1253	1568			481		
Total plutonium	308	1685	166	330	1851	2181			568		
Total (Th + U + Pu)	32,481	8790	5137	8993	13,927	22,920			2298		

Table 1. Summary characteristics of the PWR, LMFBR, and FFTF

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<sup>a</sup>CO = core, AB = axial blanket, RB = radial blanket.

<sup>b</sup>Based on full power and fuel charged.

 $c_{239}Pu + 241Pu + 239Np$ .

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The generic LMFBR is based on the Preliminary Large Breeder Reactor study.<sup>6</sup> This is a relatively conservative commercial power reactor design that produces 1250 MW(e) from a thermal power of 3956 MW(t). The core fuel is  $(Pu,U)O_2$  containing 18.5 wt % PuO<sub>2</sub>. The plutonium is assumed to be typical of that in spent LWR fuel and contains 68 wt % fissile isotopes. Both the axial and radial blankets are composed of depleted  $UO_2$ . The core and axial blanket fuels are contained in the same fuel elements, with the axial blanket split equally above and below the core. The entire radial blanket fuel element is composed of  $UO_2$ , and the elements have a larger diameter than the core/axial blanket elements to maximize the uranium loading in the reactor. One-third of the core/axial blanket assemblies and one-sixth of the radial blanket assemblies are replaced during each annual refueling. The burnup of the core fuel alone is slightly over 100,000 MWd/metric ton of initial heavy metal (MTIHM), but the average burnup of the total fuel discharged annually is only about 45,000 MWd/MTIHM.

The FFTF is a relatively large test reactor now operating in the Richland, Washington, area. Its function is to provide a high-energy neutron source typical of future commercial LMFBRs for irradiating material and fuel specimens. As a result of this role, the design of the FFTF is somewhat different from that of a commercial power reactor. $^{7,8}$  One primary difference is that there are numerous test positions in the core that may be empty or filled with a test specimen of some sort at any given time. An additional feature is that the reactor does not have the fertile axial and radial blankets typical of LMFBRs; it has only the plutonium-enriched core fuel. The full power of the reactor was assumed to be 362 MW(t), which does not include any contribution from the test loops. The plutonium used in the FFTF is relatively high grade (88 wt % fissile), and the (Pu,U)0<sub>2</sub> contains 25.6 wt %  $PuO_2$ . It is important to note that the plutonium used to fabricate the FFTF fuel was separated from its parent fuel about 10 years ago and thus contains a significant amount of  $2^{41}$ Am resulting from the decay of 14.4-year <sup>241</sup>Pu. The <sup>241</sup>Am will have a significant

effect on the transplutonium nuclide composition of the fuel. The fuel is expected to be irradiated for about 300 d, resulting in a burnup of about 45,000 MWd/MTIHM.

2.2 Description of Spent Fuel Assemblies

A summary physical description of the spent fuel assemblies from each of the three generic reactors is given in Table 2 (see Table 1 for the summary nuclear description). Because this description is substantially self-explanatory, only a few points will be elaborated on here. Photographs of the PWR and LMFBR fuel assemblies are given in Ref. 2.

It is necessary that the fuel elements be spaced apart in the fuel assembly to allow the coolant (water or liquid sodium) to circulate. This is accomplished in the PWR by the use of several grid spacers along the length of the fuel element. The grid spacer is basically a crosshatched, rectangular piece of Inconel metal that holds each of the rods in a rectangular array and has small spring clips to prevent vibration. On the other hand, the fuel element spacing in the LMFBR is accomplished by wrapping stainless steel wire in a spiral fashion around the fuel element. The elements thus find themselves closely packed in a triangular geometry with the spacing determined by the thickness of the wire wrap.

Another notable difference between the PWR and the fast reactors is that the latter use a fuel channel, a hexagonal stainless steel sheet surrounding the array of fuel elements, to prevent the coolant from moving laterally into another fuel assembly. This feature is necessary in the fast reactors to ensure that each assembly receives adequate coolant, whereas in the PWR this feature is not required.

As noted previously, because the FFTF is a test reactor, it does not have a fertile axial blanket. However, in the interest of neutron economy an axial Inconel reflector has been placed above and below the core fuel in each fuel element to reflect some of the neutrons back into the core region. This metal accounts for a significant fraction of the SMW from the FFTF fuel assembly.

		LMFBR	L	·
	PWR	Core/axial blanket	Radial blanket	FFTF
Assembly component lengths, cm			λ.	
Upper-end hardware	13	91	91	30
Gas plenum	16	191	191	107
Upper axial blanket		33		17a
Core or radial blanket	366	122	188	91
Lower axial blanket		33		27a
Lower-end hardware	11	102	102	94
Overall total	406	572	572	366
Fuel element total	385	379	370	260
idei element total	505	573	3/9	242
Assembly shape	Squa re	Hexagonal	Hexagonal	Hexagona1
Assembly flats, cm	21.4	13.78	13.78	11.62
Fuel element arrangement	Squa re	Triangular	Triangular	Triangular
Fuel elements/assembly	264	271	91	217
Fuel element OD, cm	0.95	0.650	1.270	0.584
Fuel pellet OD, cm	0.410	0.573	1.180	0.508
Fuel element pitch, cm	1.26	0.795	1.369	0.726
Cladding thickness, cm	0.058	0.030	0.038	0.038
Channel thickness, cm	<b>5</b>	0.221	0.221	0.305
Channel height, cm	1. A. A.	495	495	~300
Circumscribed volume/assembly, m <sup>3</sup>	0.186	0.114	0.114	0.052
Heavy metal/assembly, kg	461.4	117.7	172.7	33.2
MO <sub>2</sub> /assembly, kg <sup>b</sup>	523.4	133.4	195.9	37.6
Zircaloy/assembly, kg	108.4	,		
Stainless steel/assembly, kg	19.0	115.1	97.5	125.7
Inconel/assembly, kg	5.9			8.9
Assembly total weight, kg	657.9	257.7	300.3	172.2

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Table 2. Physical characteristics of PWR, LMFBR, and FFTF fuel assemblies

<sup>a</sup>UO<sub>2</sub> insulator pellet and Inconel 600 reflector.

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<sup>b</sup>(Pu,U)0<sub>2</sub> or U0<sub>2</sub>.

Finally, the fact that the PWR has Zircaloy cladding as opposed to the stainless steel cladding of the LMFBR and FFTF has an important impact on the tritium inventory in the spent fuel. The chemical properties of the Zircaloy result in all of the tritium being contained within the fuel element (except for the small fraction of leaking elements). On the other hand, stainless steel does not have these characteristics; as a result, about 90% of the tritium produced in the fuel is expected to diffuse through the stainless steel cladding into the sodium coolant, from which it will be recovered. The tritium inventories in this document reflect this difference; i.e., the LMFBR and FFTF spent fuels contain only 10% of the tritium produced during irradiation.

2.3 Description of High-Level Waste and Structural Material Waste

The HLW and SMW are both produced as a result of chemically reprocessing spent fuel to recover the economic values (plutonium and uranium). The first part of the reprocessing sequence involves chopping the spentfuel assembly with a heavy-duty shear, cutting the fuel elements into segments a few centimeters in length. The segments are then immersed in concentrated nitric acid to dissolve the oxide fuel matrix while leaving the fuel assembly structural materials essentially intact. At this point the volatile elements are freed, escaping from the dissolver into the off-gas treatment system. After dissolution is complete, the structural materials are removed, becoming the SMW described below. The nitric acid solution containing the dissolved spent fuel is contacted with tributyl phosphate, which removes the uranium and plutonium for separation and purification further downstream in the plant. The fission products and actinides other than uranium and plutonium constitute liquid HLW.

After its generation the liquid HLW will most likely be concentrated by evaporation, then heated to a high temperature to drive off the nitric acid and convert the dissolved fission product and actinide nitrates to oxides (i.e., calcined), and then incorporated into a monolith of glass (i.e., vitrified). The SMW may be compacted to reduce its volume.

Then it will most likely be combined with an inert solid to counteract the pyrophoric nature of the fine Zircaloy particles produced during shearing or incorporated into a concrete matrix. It should be noted that there are many alternative treatment technologies for the HLW and SMW; those mentioned above are only the most likely at present.

As noted above, the HLW contains all of what is in the spent-fuel oxide except the volatile elements and the economic values. More specifically, it is the contents of the spent fuel, decayed until the time of reprocessing, with the following species removed: 99.5% of the uranium and plutonium, 99.9% of the halogen elements (principally iodine), and 100% of the tritium, <sup>14</sup>C, and noble gases (krypton, xenon, radon). In addition, because stainless steel is corroded somewhat by the nitric acid, it is assumed that 0.69% of the LMFBR and FFTF fuel assembly structural material is in the HLW.<sup>9</sup>

The SMW principally comprises all of the hardware that constitutes the fuel assembly except the fuel material itself. Thus, for the PWR this is predominantly Zircaloy with some stainless steel and Inconel. For LMFBR and FFTF it is virtually all stainless steel, with some Inconel from the reflector in the case of the FFTF. In addition, it is assumed that 0.05% of the nonvolatile components of the spent-fuel oxide (i.e., the HLW plus the uranium and plutonium) are included in the SMW, a result of the shearing operation, which pinches the end of some of the cladding segments closed so that the fuel matrix is inaccessible to the nitric acid in the dissolver. It is also assumed that 30% of the tritium produced in the PWR fuel is present in the SMW as a part of the Zircaloy cladding.

The time necessary for the spent fuel to be reprocessed, relative to its discharge from the reactor, is assumed to be 160 d for the PWR and FFTF and 90 d for the LMFBR.

#### 2.4 Comparisons and Conversions

One comparison that is often made is that of the toxicity of some material with the toxicity of uranium ore. In the recent past this has

most often been done in studies involving LWRs, which include the PWRs employed here. The basis of the comparison has been to compare one of the fuel cycle materials with the toxicity of the uranium ore mined to produce that material. If the total losses of uranium at the beginning of the fuel cycle are assumed to be 12% (10% during milling, 1% during conversion, and 1% during fabrication) and if the enrichment plant tails are assumed to be 0.25%, then 7.16 MT of uranium must be mined to produce 1.0 MTIHM of fresh fuel. This amount of uranium ore, when in equilibrium with its shorter-lived daughters (i.e., when it is most toxic), requires 1.1 x  $10^8$  m<sup>3</sup> of water to dilute it to the standards given in Appendix B, Table II of 10 CFR 20.10 On other occasions, the comparison of the toxicity of uranium ore to that of a fuel cycle material is made on a volumetric basis. The volumetric toxicity of carnotite ore, virtually the only source of uranium today, is about  $10^5 \text{ m}^3$  of water/m<sup>3</sup> of ore. The toxicity of pitchblende, a very-highgrade uranium ore, is about  $10^8 \text{ m}^3$  of water/m<sup>3</sup> of ore.

The conversion most often of interest is that from a MTHM basis to an electrical basis. Most often the electrical unit used is the GWy(e), although it is usually not clear whether the "GW" in this unit is based on reactor capacity or electricity actually produced by the reactor. To circumvent this problem, Table 3 contains the conversion factors on both a capacity and production basis for the PWR and LMFBR. Although these factors are meaningless for the FFTF because it does not produce electricity, they have been included in the table for the case when it is desirable to put the FFTF on an equivalent basis with other reactors. The thermal efficiency of the FFTF has been assumed to be the same as that of the LMFBR. It should be noted that values at two thermal efficiencies are given for both the LMFBR and the FFTF because it may be desirable to use a value of 40%, which is more typical of current designs, rather than the very conservative 31.6% used in the original design.

· · · · · · · · · · · · · · · · · · ·			Thermal effic:	lency = $31.6\%$	Thermal effic	ciency = 40%
Parameter		PWR	LMFBRa	FFTF	LMFBRa	FFTF
MTIHM/GWy(e) of read	tor capacity		<u> </u>			· · ·
Capacity factor:	0.5	16.81	12.76	12.84	10.08	10.14
	0.6	20.17	15.31	15.40	12.09	12.17
	0.7	23.54	17.86	17.97	14.11	14.20
· ·	0.8	26.90	20.41	20.54	16.12	16.23
	0.9	30.26	22.97	23.10	18.15	18.25
	1.0	33.62	25.52	25.67	20.16	20.28
MTIHM/GWy(e) of elec produced	tricity	33.62	25.52	25.67	20.16	20.28

Table 3. Factors for converting MTIHM to GWy(e)

<sup>a</sup>For use with blended fuel or wastes. For core fuel alone, multiply factors by 0.447; for core and axial blanket, multiply factors by 0.663; for radial blanket, multiply factors by 6.24.

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#### 3. DESCRIPTION OF GRAPHICAL AND SUMMARY TABULAR FUEL CYCLE MATERIAL CHARACTERISTICS

By applying the computer codes ORIGEN2 and ORMANG to the generic reactors and fuel cycle materials described in the preceding sections, graphical and summary tabular information concerning the radioactivity, thermal power, and toxicity of the materials has been generated. This information is contained in the appendixes to this report. The first three appendixes contain information for each of the three generic reactors, with each graph or table presenting a total and the principal contributing nuclides as a function of decay time. The last two appendixes contain only graphs, which have curves for the totals for all materials for a single reactor or for a single material for all reactors (i.e., the last two appendixes compare materials or reactors).

It should be noted that the time scale on all of the graphs and tables is relative to the time at which the spent fuel is discharged from the reactor. Thus, the curves for HLW and SMW will be offset from the vertical axis by a time increment equal to the lag time between discharge and reprocessing. The time span considered in all graphs and tables is 0.1 to  $10^6$  years. The vertical axis is always seven logarithmic decades, with the maximum selected so that none of the curves are off scale at the top.

#### 3.1 Description of Graphs and Tables Giving Totals and Principal Contributing Nuclides

Appendixes A-C contain the fuel cycle material characteristics in graphical and tabular form for the PWR, LMFBR, and FFTF, respectively. The information presented in the graphs consists of the total for a particular characteristic as a function of decay time (e.g., total radioactivity of spent PWR fuel) and the decay curves for the nuclides that are the principal contributors to this total. The principal contributors are determined by dividing the amount of each nuclide (in appropriate units) at a given time by the total at that time, summing this fraction over a list of times specified as input to the ORMANG

code and selecting the nuclides with the largest summation. In the present case the times used in this determination were every logarithmic decade (e.g., 0.1 year, 1 year, 10 years, etc.) over the time span 0.1 to  $10^6$  years inclusive. The graphs are limited such that no more than 14 principal contributors can be plotted on any single graph.

The nuclides in the summary tables are determined using the same test as that described above for the graphs. However, in the tables the first 23 contributors are included.

It should be noted that there are two general ways in which a nuclide can become a principal contributor. In the first way the nuclide is very significant at one or two of the times tested; i.e., the nuclide contributes nearly 100% to the total for a short time. In the second way a nuclide contributes a much smaller fraction to the total but does so over a much longer time span. In many of the graphs an example of the former is  $^{144}$ Ce and an example of the latter is  $^{237}$ Np. As a result, it is occasionally possible that a nuclide will be listed in the tabular summary but not appear on a graph because it is below scale.

Table 4 has been included to facilitate the location of the graphs and tables giving the totals and principal contributors. Given the desired reactor, fuel cycle material, and characteristic, the reader can readily determine from Table 4 the appropriate graph or table. The graphs and summary tables appear together in Appendixes A-C.

3.2 Description of Graphs Comparing Material and Reactor Totals

Appendixes D and E contain graphs comprising the totals for all fuel cycle materials from one reactor on a single graph (e.g., thermal power of spent fuel, HLW, and SMW from the FFTF on one graph) and the totals for a single fuel cycle material from all three reactors on one graph (e.g., total toxicity of HLW from the PWR, LMFBR, and FFTF on one plot), respectively. The principal contributing nuclides and summary tables are not given in these appendixes since the information is available in Appendixes A-C.

	Characteristic in graph or table								
Reactor/material in graph or table	Radioactivity (Ci)	Thermal power (W)	Toxicity (m <sup>3</sup> of water)						
PWR									
Spent fuel	A.1	A.4	A.7						
HLW	A.2	A.5	A.8						
SMW	A.3	A.6	A.9						
LMFBR									
Spent core + axial blanket fuel	B.1	B.6	B.11						
Spent radial blanket fuel	B.2	B.7	B.12						
Spent core fuel	B.3	B.8	B.13						
HLW	B.4	B.9	B.14						
SMW	B.5	B.10	B.15						
FFTF									
Spent fuel	C.1	C.4	C.7						
HLW	C.2	C.5	C.8						
SMW	C.3	C.6	C.9						

Table 4. Numbers for graphs and summary tables giving totals and principal contributing nuclides

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Table 5 has been included to facilitate the location of the comparison of graphs in Appendixes D and E.

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Table	5.	Numbers reactors	for and	graphs giving comparisons of fuel cycle materials	

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	Materia	al characteristic i	in graph
Contents of one graph	Radioactivity (Ci)	Thermal power (W)	Toxicity (m <sup>3</sup> of water)
Totals for all fuel cycle materials from the following reactor:			
PWR	D.1	D•4	D.7
LMFBR	D.2	D.5	D.8
FFTF	D.3	D.6	D.9
Totals for all reactors from the following fuel cycle material:	· .		
Spent fuel	E.1	E.4	E.7
HLW	E.2	<b>E.5</b>	E.8
SMW	E.3	E.6	E.9

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#### 4. REFERENCES

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Appendix A. CHARACTERISTICS OF PWR SPENT FUEL, HIGH-LEVEL WASTE, AND STRUCTURAL MATERIAL WASTE

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Appendix A.1. Radioactivity of PWR Spent Fuel, High-Level Waste, and Structural Material Waste

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Fig. A.l. Radioactivity of PWR spent fuel as a function of decay time.

	Radioactivity (Ci/MTIHM)																			
Time – (years)	TOTAL		PU239.		C\$137	* ~ *	SR 90		PU240		AM241		PR144	_	CE144	PU241	TC 99	ZR 93	<u> </u>	RH106
$\begin{array}{c} 1.000E-01\\ 3.000E-01\\ 5.000E-01\\ 1.000E 00\\ 2.000E 00\\ 3.000E 01\\ 3.000E 01\\ 1.000E 01\\ 3.000E 01\\ 3.000E 01\\ 3.000E 02\\ 1.000E 03\\ 3.000E 04\\ 1.000E 04\\ 3.000E 04\\ 3.000E 04\\ 3.000E 04\\ 3.000E 04\\ 1.000E 05\\ 3.000E 06\\ 05\\ 000E 06\\ 00\\ 00\\ 00\\ 00\\ 00\\ 00\\ 00\\ 00\\ 00\\$	1.173E 6.334E 2.507E 1.386E 2.507E 1.386E 3.935E 2.808E 2.1296E 4.280E 1.755E 2.808E 2.1296E 4.280E 1.697E 5.524E 3.433E 3.433E	0766666555554332222111	3.131E 3.131E 3.131E 3.131E 3.131E 3.131E 3.131E 3.129E 3.129E 3.129E 3.129E 3.129E 3.105E 2.884E 2.375E 1.3475E 1.3475E 1.3475E 3.589E	00000000000000000000000000000000000000	$\begin{array}{c} 2.025E \\ 2.015E \\ 1.0982E \\ 1.9982E \\ 1.8938E \\ 1.8938E \\ 1.894E \\ 1.611E \\ 1.015E \\ 2.013F \\ 1.9982E \\ 1.5982E \\ 1.598E \\ 0.0 \\ 0.0 \\ 0.0 \\ 0.0 \\ 0.0 \\ 0.0 \end{array}$	000000000000000000000000000000000000000	$\begin{array}{c} 1.459E \\ 1.452E \\ 1.445E \\ 1.362E \\ 1.395E \\ 1.395E \\ 1.3299E \\ 1.153E \\ 9.086E \\ 7.162E \\ 1.153E \\ 1.354E \\ 1.354E \\ 1.354E \\ 1.354E \\ 1.425E \\ 0.0 $	00000000000000000000000000000000000000	5.259E 5.259E 5.259E 5.259E 5.261E 5.261E 5.264E 5.264E 5.264E 5.264E 5.264E 5.264E 1.367E 2.272E 5.1367E 2.272E 5.1367E 2.272E 5.1367E 2.200 2.13657E 2.272E 5.264E 2.272E 5.264E 2.272E 5.264E 2.272E 5.2659E 5.2559E 5.2659	00000000000000000000000000000000000000	1.226E 1.626E 2.025E 2.052E 2.	222222222222222222222222222222222222222	$\begin{array}{c} 1 \bullet 15 \ 3E & 0 \\ 9 \bullet 64 \ 5E & 0 \\ 8 \bullet 072E & 0 \\ 8 \bullet 072E & 0 \\ 2 \bullet 122E & 0 \\ 8 \bullet 709E & 0 \\ 1 \bullet 86 \ 7E & 0 \\ 1 \bullet 708E & 0 \\ 1 \bullet 708E & 0 \\ 0 \bullet 3 \bullet 136E - 0 \\ 0 \bullet 0 \\ \end{array}$	6555544226	$\begin{array}{c} 1 \cdot 153E & 96\\ 9 \cdot 645E & 05\\ 8 \cdot 072E & 05\\ 5 \cdot 170E & 05\\ 8 \cdot 122E & 05\\ 8 \cdot 709E & 04\\ 1 \cdot 467E & 04\\ 1 \cdot 709E & 02\\ 2 \cdot 314E - 02\\ 3 \cdot 136E - 06\\ 0 \cdot 0\\ 0 \cdot 0\\$	$\begin{array}{c} 1.254E 05\\ 1.242E 05\\ 1.230E 05\\ 1.201E 05\\ 1.901E 05\\ 1.901E 05\\ 9.908E 04\\ 4.813E 04\\ 4.813E 04\\ 4.813E 04\\ 1.023E 03\\ 8.770E-02\\ 1.628E-02\\ 1.628E-02\\ 1.801E-03\\ 5.970E-06\\ 4.908E-13\\ 0.0\end{array}$	1.307E 01 1.307E 01 1.307E 02 1.307E 02 1.306E 02 1.266E 02 1.306E	$\begin{array}{c} 1.924E \\ 1.940E \\ 1.956E \\ 1.956E \\ 2.972E \\ 2.144E \\ 2.277E \\ 2.277E \\ 3.512E \\ 3.5521E \\ 3.5521E \\ 3.5521E \\ 3.5526E \\ 3.5536E \\ 3.356E \\ 3.356E \\ 3.366E \\ 3.074E \\ $		5.433E 05 4.735E 05 2.925E 05 1.471E 05 1.869E 04 1.869E 04 6.005E 02 6.919E-04 8.614E-25 0.0 0.

Table A.1. Radioactivity of PWR spent fuel as a function of decay time

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Radioactivity (Ci/MTIHM)

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Time												
(years)	RU106	NB 95	ZR 95	PM147	R0103	C\$134	¥ 91	KHI03M	NP237	PA233	CEIGI	0234
1.300E-01 3.000E-01 5.000E-01 1.000E 00 2.000E 00 3.000E 00 5.000E 00 1.000E 01 2.000E 01	5.433E 05 4.735E 05 4.126E 05 2.923E 05 1.471E 05 7.396E 04 1.869E 04 6.005E 02 6.369E-01 4.005E 02	1.494E 06 8.872E 05 4.518E 05 6.850E 04 1.384E 03 2.652E 01 9.361E-03 2.392E-11 1.563E-28	1.110E 06 5.032E 05 2.280E 05 3.153E 04 6.031E 02 1.153E 01 4.217E-03 1.078E-11 7.043E-29	1.334E 05 1.273E 05 1.208E 05 8.124E 04 6.239E 04 3.677E 04 9.814E 03 6.989E 02	8.653E 05 2.3E4E 05 6.570E 04 2.618E 03 4.204E 00 6.694E-03 1.689E-08 1.709E-22 0.0	1.512E 05 1.414E 05 1.322E 05 1.117E 05 7.983E 04 5.704E 04 2.912E 04 5.422E 03 1.881E 02	7.506E 05 3.159E 05 1.330E 04 2.019E 02 2.665E 00 4.648E-04 1.866E-13 3.006E-32	7.801E 05 2.149E 05 5.922E 04 2.360E 03 3.790E 00 6.033E-03 1.523E-08 1.541E-22 0.0	3.120E-01 3.123E-01 3.123E-01 3.123E-01 3.123E-01 3.123E-01 3.13E-01 3.131E-01 3.153E-01 3.224E-01 3.20E-01	3.040E-01 3.110E-01 3.120E-01 3.123E-01 3.123E-01 3.123E-01 3.13E-01 3.131E-01 3.224E-01 3.224E-01	7.639E 05 1.609E 05 3.391E 04 6.906E 02 2.867E-01 1.190E-04 2.050E-11 2.525E-28 0.0	1.123E 00 1.124E 00 1.129E 00 1.129E 00 1.136E 00 1.136E 00 1.156E 00 1.188E 00 1.248E 00
1.000E 02	8.614E-25	0.0	-0.0	4.623E-07	0.0	3.933E-10	0.0	0.0	4.163E-01	4.163E-01	0.0	1.594E 00
3.000E 02	0.0	0.0	0.0	0.0	0.0	0.0	0.0	3.0	6.260E-01	6.260E-01	0.0	1.906E 00
1.000E 03	0.0	0.0	0.0	0.0	0.3	0.0	0.0	0.0	1.000E 00	1.000E 00	5.5	1.984E 00
3.000E 03	0.0	0.0	v.0	0.0	0.0	0.0	0.0	0.0	1.173E 00	1.173E 00	0.0	1.975E 03
3.0005 04	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	1.170E 00	1.170E 00	0.0	1.9426 00
1.000E 05	0.0	0.0	ŏ.ŏ	0.0	0.0	0.0	0.0	0.0	1.144E 00	1.144E 00	ě.5	1.576E 00
3.000E 05	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	1.372E 00	1.072E 00	C.0	1.331E 03
1.000E 00	Uay	V• V	0.0	0.0	0.0	0.9	U + U	0.0	0.34./E=UL	0.34/2-01	0.0	4.1305-01



Fig. A.2. Radioactivity of PWR high-level waste as a function of decay time.

					Radio	pactivity (C	i/MTIHM)					
Time	TOTAL	CS137	TC 99	SR 90	ZR 93	AM 24 1	PR144	CE144	AM243	NP239	RH106	RU106
1.000E-01 3.000E-01 1.000E 00 2.000E 00 3.000E 00 1.000E 01 1.000E 01 1.000E 01 1.000E 02 1.000E 03 1.000E 04 3.000E 05 1.000E 05	3.922E 36 3.027E 06 1.723E 06 1.723E 06 1.723E 06 6.777FE 05 4.300E 05 2.9775E 05 3.382E 05 3.382E 02 1.736E 05 3.382E 02 1.736E 05 3.382E 02 1.736E 05 3.382E 02 1.736E 05 3.903E 01 2.431E 01 1.675E 01 1.181E 01 6.140E 00	$\begin{array}{c} 2.004E 05\\ 1.995E 05\\ 1.995E 05\\ 1.962E 05\\ 1.962E 05\\ 1.918E 05\\ 1.789E 05\\ 1.594E 05\\ 1.594E 05\\ 1.265E 05\\ 1.904E 05\\ 1.963E 02\\ 1.856E 05\\ 1.963E 02\\ 1.856E - 05\\ 1.582E - 25\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.$	$\begin{array}{c} 1.307E & 01\\ 1.306E & 01\\ 1.306E & 01\\ 1.206E & 01\\ 1.265E & 01\\ 1.265E & 01\\ 1.485E & 00\\ 4.923E & 00\\ 5.047E-01\\ \end{array}$	1.444E 05 1.437E 05 1.437E 05 1.413E 05 1.380E 05 1.387E 05 1.347E 05 1.347E 05 1.347E 05 1.412E 02 6.659E-06 1.410E-26 0.0 0.0 0.0	1.958E 00 1.974E 00 2.028E 00 2.102E 00 2.303E 00 2.577E 00 3.5180E 00 3.519E 00 3.519E 00 3.519E 00 3.514E 00 3.514E 00 3.514E 00 3.514E 00 3.514E 00 3.577E 00 2.238E 00	1.900E 02 1.901E 02 1.902E 02 1.905E 02 1.912E 02 1.917E 02 1.927E 02 1.927E 02 1.964E 02 1.964E 02 1.964E 02 1.964E 02 1.964E 02 1.964E 02 1.9797E 02 1.305E 02 4.249E 01 1.735E 00 9.220E-03 1.800E-03 5.975E-06 5.187E-13 J.0	7.800E 05 6.527E 35 5.463EE 05 1.436E 05 1.436E 03 1.156E 03 1.156E 02 2.15266E-02 2.122E-06 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	7.799E 05 6.527E 05 5.462E 05 3.499E 05 1.436E 05 5.483E 04 9.925E 03 1.155E 02 2.122E-06 0.0 0.	1.706E 01 1.706E 01 1.706E 01 1.706E 01 1.706E 01 1.706E 01 1.703E 01 1.703E 01 1.703E 01 1.703E 01 1.659E 01 1.659E 01 1.659E 01 1.659E 01 1.659E 01 1.659E 01 1.659E 01 3.652E 08 3.678E 08	1.707E 01 1.706E 01 1.706E 01 1.706E 01 1.706E 01 1.705E 01 1.705E 01 1.705E 01 1.703E 01 1.703E 01 1.659E 01 1.659E 01 1.659E 01 1.659E 01 1.659E 01 1.659E 01 1.659E 01 1.659E 01 3.671E 00 1.461E-03 3.692E-08 3.578E-08	4.019E 05 3.502E 05 3.5502E 05 2.164E 05 5.471E 04 4.442E 05 5.915E-01 4.752E-04 5.915E-25 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.	4.019E 05 3.502E 05 3.552E 05 2.164E 05 5.471E 04 4.442 92 4.560E-01 4.752E-04 5.915E-25 0.0 0.0 0.0 0.0 0.0 0.0 0.0
Radioactivity (Ci/MTIHM)												
(years)	PU239	PU240	PM147	NB 95	CS134	NP 237	PA233	CS135	ZR 95	U233.	TH229	40225
1.000E-01 3.000E-01 1.000E 00 2.000E 00 3.000E 00 5.000E 00 1.000E 01 3.000E 01 3.000E 01 3.000E 02 3.000E 02 1.000E 03 1.000E 04 1.000E 04 1.000E 05 3.000E 05 1.000E 06	1.565E 00 1.565E 00 1.565E 00 1.567E 00 1.567E 00 1.567E 00 1.5787E 00 1.5787E 00 1.5787E 00 1.5787E 00 1.5787E 00 1.6287E 00 1.6299E 00 3.894E 00 3.894E 00 3.894E 00 3.878E-08	2.645E 00 2.676E 00 2.738E 00 2.738E 00 3.935E 00 3.935E 00 3.947E 00 4.843E 00 5.452E 00 6.583E 00 6.583E 00 6.583E 00 6.583E 00 6.583E 00 2.354E 00 2.8254E 00 2.972E 00 3.060E 00 2.972E 0	1.195E 05 1.134E 05 9.421E 04 7.234E 04 5.554E 04 3.274E 04 8.734E 03 6.222E 02 4.431E 01 4.431E 01 4.431E 01 0.0 0.0 0.0 0.0 0.0 0.0 0.0	$\begin{array}{c} 3.939E  05\\ 1.875E  05\\ 8.708E  04\\ 1.230E  04\\ 2.44685E  00\\ 1.653E-03\\ 4.226E-12\\ 2.761E-29\\ 1.804E-46\\ 0.0\\ 0.3\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0$	$\begin{array}{c} 1 \cdot 305E  05\\ 1 \cdot 220E  05\\ 1 \cdot 140E  05\\ 9 \cdot 639E  04\\ 4 \cdot 921E  04\\ 2 \cdot 512E  04\\ 1 \cdot 622E  03\\ 1 \cdot 622E  02\\ 5 \cdot 627E  00\\ 3 \cdot 394E - 10\\ 0 \cdot 0\\ 0 \cdot 0\\$	$\begin{array}{c} 3.121E-01\\ 3.121E-01\\ 3.121E-01\\ 3.121E-01\\ 3.122E-01\\ 3.122E-01\\ 3.122E-01\\ 3.122E-01\\ 3.122E-01\\ 3.123E-01\\ 3.133E-01\\ 3.140E-01\\ 3.459E-01\\ 3.459E-01\\ 3.535E-01\\ 3.535E-01\\ 3.535E-01\\ 3.218E-01\\ 3.21$	$\begin{array}{c} 3 \cdot (19E-01) \\ 3 \cdot (121E-01) \\ 3 \cdot (121E-01) \\ 3 \cdot (121E-01) \\ 3 \cdot (122E-01) \\ 3 \cdot (122E-01) \\ 3 \cdot (123E-01) \\ 3 \cdot (124E-01) \\ 3 \cdot (133E-01) \\ 3 \cdot (132E-01) \\ 3 \cdot (132$	3.449E-01 3.438E-01 3.458E-01	1.961E 05 8.897E 04 4.328E 04 5.579E 03 1.3658C 02 2.037E 00 7.446E-04 1.903E-12 1.244E-29 8.125E-47 0.3 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	$\begin{array}{c} 2 & 0.76E-0.7\\ 4 & 805E-0.7\\ 7 & 534E-0.6\\ 2 & 946E-0.6\\ 4 & 457E-0.6\\ 2 & 946E-0.6\\ 4 & 457E-0.6\\ 1 & 417E-0.5\\ 2 & 785E-0.5\\ 1 & 383E-0.4\\ 4 & 213E-0.4\\ 4 & 213E-0.4\\ 1 & 4555E-0.3\\ 1 & 5505E-0.3\\ 1 & 505E-0.3\\ 1 & 505E-0.3$	5.080E-08 5.081E-08 5.087E-08 5.087E-08 5.138E-08 5.1255E-08 5.763E-08 7.748E-08 7.748E-08 7.748E-08 7.748E-08 7.023E-07 5.918E-06 6.564E-05 5.751E-04 5.316E-03 2.933E-02 1.127E-01 2.728E-01 2.728E-01	5.085E-08 5.081E-08 5.081E-08 5.08E-08 5.108E-08 5.125E-08 5.763E-08 7.748E-08 7.748E-08 7.748E-08 7.102E-07 7.023E-07 5.918E-06 6.564E-05 5.751E-04 5.316E-03 2.931E-02 1.127E-01 2.469E-01 2.728E-01

Table A.2. Radioactivity of PWR high-level waste as a function of decay time

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Fig. A.3. Radioactivity of PWR structural material waste as a function of decay time.

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Time       Time         (years)       TOTAL       NI 59       NI 63       CO 60       ZR 93       NB 93M       FE 55       NB 95       NB 94       SN119M       ZR 95       C 14         1.000E-01       4.009E       04       5.181E       00       6.580E       02       7.232E       03       1.269E-01       9.850E-03       4.857E       03       1.263E       00       3.802E       03       5.780E       03       9.542E-01         3.000E-01       2.811E       04       5.181E       00       6.561E       02       7.45E       03       1.269E-01       1.097E-02       4.6605E       03       5.526E       03       1.283E       00       3.092E       03       2.620E       03       9.542E-01         1.000E-01       2.189E       04       5.181E       00       6.561E       02       6.425E       03       1.269E-01       1.208E-02       4.366E       03       1.263E       00       1.500E       03       1.642E       02       9.541E-01         2.000E       00       1.656E       04       5.181E       00       6.536E       03       1.269E-01       2.006E-02       2.927E       03       7.211E       00       1.500E	Radioactivity (Ci/MTIHM)											
1.900E-01 4.309E 04 5.181E 00 6.580E 02 7.232E 03 1.269E-01 9.850E-03 4.857E 03 1.161E 04 1.283E 00 3.802E 03 5.780E 03 9.542E-01 3.000E-01 2.811E 04 5.181E 00 6.571E 02 7.045E 03 1.269E-01 1.097E-02 4.605E 03 5.526E 03 1.283E 00 3.092E 03 2.620E 03 9.541E-01 5.000E-01 2.189E 04 5.181E 00 6.561E 02 6.862E 03 1.269E-01 1.208E-02 4.366E 03 2.567E 03 1.283E 00 2.515E 03 1.167E 03 9.541E-01 1.000E 00 1.566E 04 5.181E 00 6.536E 02 6.425E 03 1.269E-01 1.481E-02 3.821E 03 3.625E 02 1.283E 00 1.500E 03 1.6642E 02 9.541E-01 2.000E 00 1.165E 04 5.181E 00 6.487E 02 5.633E 03 1.269E-01 2.006E-02 2.927E 03 7.211E 00 1.283E 00 1.500E 03 1.6642E 02 9.541E-01 3.000E 00 9.476E 03 5.181E 00 6.438E 02 4.939E 03 1.269E-01 2.505E-02 2.422E 03 1.381E-01 1.283E 00 1.900E 02 6.003E-02 9.538E-01 5.000E 01 3.405E 03 5.181E 00 6.438E 02 3.796E 03 1.269E-01 3.430E-02 1.315E 03 4.873E-05 1.282E 00 2.405E 01 2.195E-05 9.536E-01 1.000E 01 3.405E 03 5.181E 00 6.107E 02 1.967E 03 1.269E-01 5.370E-02 2.412E 01 8.138E-13 1.282E 00 2.405E 01 2.195E-05 9.536E-01 2.000E 01 1.352E 03 5.180E 00 5.664E 02 5.279E 02 1.229E-01 8.037E-02 2.412E 01 8.138E-31 1.282E 00 4.479E-06 3.656E-31 9.519E-31 3.000E 31 8.258E 02 5.180E 00 5.253E 02 1.417E 02 1.268E-01 9.640E-02 1.677E 00 5.317E-48 1.281E 00 1.458E-10 2.395E-48 9.507E-01 3.000E 02 3.388E 02 5.187E 00 5.253E 02 1.417E 02 1.268E-01 9.640E-02 1.677E 00 5.317E-48 1.281E 00 1.458E-10 2.395E-48 9.507E-01 3.000E 31 8.258E 02 5.180E 00 5.253E 02 1.417E 02 1.268E-01 9.640E-02 1.677E 00 5.317E-48 1.281E 00 1.458E-10 2.395E-48 9.507E-01 3.000E 31 8.258E 3.5180E 3.5180E 3.5180E 3.5180E 3.5180E 3.5180E 3.5180E 3.5180E 3.500E-3.5180E 3.500E-3.5180E 3.500E-3.5180E-3.500E-3	Time — (years)	TOTAL	NI 59	NI 63 CO 60	ZR 93	NB 93M	FE 55	NB 95	NB 94	SN119M	ZR 95	C 14
3.000E 02 7.844E 01 5.168E 00 6.870E 01 5.342E-14 1.268E-01 1.205E-01 0.0 0.0 1.270E 00 0.0 0.0 9.202E-01 1.000E 03 8.709E 00 5.137E 00 3.516E-01 0.0 1.268E-01 1.205E-01 0.0 0.0 1.440E 00 0.0 C.J 8.455E-01 3.000E 03 7.513E 00 5.048E 00 1.005E-07 0.0 1.268E-01 1.205E-01 0.0 0.0 1.158E 00 0.0 C.J 8.455E-01 1.000E 04 6.429E 00 4.751E 00 0.0 0.0 1.263E-01 1.200E-01 0.0 0.0 9.116E-01 0.0 0.0 2.846E-01 3.000E 04 4.821E 00 3.995E 00 0.0 0.0 1.251E-01 1.189E-01 0.0 0.0 4.655E-01 0.0 0.0 2.531E-02 1.000E 05 2.484E 00 2.178E 00 0.0 0.0 1.212E-01 1.152E-01 0.0 0.0 4.546E-05 0.0 0.0 5.313E-02 3.000E 05 6.183E-01 3.851E-01 0.0 0.0 1.212E-01 1.152E-01 0.0 0.0 0.0 4.546E-05 0.0 0.0 5.313E-02 1.647E-16	$\begin{array}{c} 1.000E-01\\ 3.000E-00\\ 1.000EE-00\\ 3.000EE-00\\ 3.000E-00\\ 3.000E-0$	$\begin{array}{c} 4.009E & 04\\ 2.819E & 04\\ 1.566E & 04\\ 1.566E & 04\\ 1.566E & 04\\ 3.476E & 03\\ 3.405E & 03\\ 3.405E & 03\\ 3.405E & 03\\ 3.405E & 02\\ 3.388E & 02\\ 7.513E & 01\\ 6.7513E & 01\\ 6.429E & 00\\ 6.429E & 00\\ 6.483E & 01\\ 6.183E & 01\\ \end{array}$	5.181E 00 5.181E 00 5.181E 00 5.181E 00 5.181E 00 5.181E 00 5.181E 00 5.181E 00 5.181E 00 5.180E 00 5.180E 00 5.180E 00 5.168E 00 5.168E 00 5.168E 00 5.168E 00 5.168E 00 5.178E 00 3.955E 00 3.955E 00 3.851E-01	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	9.850E-03 1.097E-02 1.208E-02 2.006E-02 2.505E-02 3.430E-02 9.640E-02 1.198E-01 1.205E-01 1.205E-01 1.205E-01 1.205E-01 1.189E-01 1.152E-01 1.152E-01	$\begin{array}{c} 4.857E 03\\ 4.605E 03\\ 3.821E 03\\ 2.927E 03\\ 2.927E 03\\ 3.468E 02\\ 2.442E 03\\ 1.315E 03\\ 3.468E 02\\ 2.412E 01\\ 1.677E 01\\ 1.318E-08\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0$	$\begin{array}{c} 1.161E & 04\\ 5.526E & 03\\ 2.567E & 03\\ 3.625E & 02\\ 7.211E & 00\\ 1.381E-01\\ 4.873E-05\\ 1.246E-13\\ 8.138E-31\\ 5.317E-48\\ 3.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0$	$\begin{array}{c} 1.283 \pm 00\\ 1.282 \pm 00\\ 1.270 \pm 00\\ 1.270 \pm 00\\ 1.270 \pm 00\\ 1.270 \pm 00\\ 0.158 \pm 00\\ 0.165 \pm 01\\ 4.615 \pm 01\\ 4.218 \pm 02\\ 4.54 \pm 0.0\\ 1.58 \pm 00\\ 1.58 \pm 0.0\\ $	3.802E 03 3.092E 03 2.51500E 03 1.500E 03 1.338E C2 1.900E 01 1.376E 01 1.376E 01 1.479E - 06 1.458E - 10 0.00	$5 \cdot 780E 03$ $2 \cdot 620E 03$ $1 \cdot 187E 02$ $3 \cdot 139E 02$ $3 \cdot 139E - 02$ $2 \cdot 195E - 05$ $5 \cdot 610E - 05$ $5 \cdot 610E - 31$ $2 \cdot 395E - 48$ $0 \cdot 3$ $0 \cdot 3$	$\begin{array}{c} 9 & 54 \\ 28 & -01 \\ 9 & 54 \\ 18 & -01 \\ 9 & 54 \\ 18 & -01 \\ 9 & 54 \\ 18 & -01 \\ 9 & 53 \\ 48 & -01 \\ 9 & 53 \\ 68 & -01 \\ 9 & 53 \\ 68 & -01 \\ 9 & 53 \\ 68 & -01 \\ 9 & 53 \\ 68 & -01 \\ 9 & 53 \\ 78 & -01 \\ 9 & -01 \\$

Table A.3. Radioactivity of PWR structural material waste as a function of decay time

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Radioactivity (Ci/MTIHM)

(years) SB12	5 CS137	AM241 SR	R 90 PU240	H 3 PU23	9 TE125M	CO 58	PR144 CI	E144	нз
$\begin{array}{cccccccccccccccccccccccccccccccccccc$	3       1.002E       02         10       03       9.977E       01         10       03       9.931E       01         10       03       9.931E       01         10       02       9.9374E       01         10       02       9.9374E       01         10       02       9.974E       01         10       02       7.974E       01         10       0.0       6.329E       01         10       0.0       9.813E-0       01         10       9.967E       01       0.0         10       9.967E       01       01         10       9.813E-0       02       9.280E-0       01         10       9.813E-0       01       02       9.280E-0       01         10       0.0       0.0       0.0       0.0       0.0	$\begin{array}{c} 1.049E-01 7.2\\ 1.244E-01 7.1\\ 1.438E-01 7.1\\ 1.914E-01 7.2\\ 2.831E-01 6.2\\ 3.704E-01 6.2\\ 1.3704E-01 6.2\\ 1.340E 00 4.2\\ 1.616E 00 3.2\\ 1.340E 00 4.2\\ 1.616E 00 3.2\\ 1.873E 00 5.2\\ 1.873E 00 5.2\\ 1.873E 00 5.2\\ 1.873E 0.2\\ 2.589E-16 0.2\\ 2.589E-16 0.2\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\$	•221E       01       2.631E-01         •187E       01       2.631E-01         •153E       01       2.631E-01         •069E       01       2.632E-01         •903E       01       2.632E-01         •741E       01       2.632E-01         •706E       01       2.636E-01         •697E       02       2.636E-01         •699E       02       2.632E-01         •706E       01       2.636E-01         •699E       02       2.622E-01         •355E-09       2.636E-01         •355E-09       2.628E-01         •355E-09       2.385E-01         •063E-30       1.929E-31         •0       9.183E-02         •0       1.102E-02         •0       2.865E-06         •0       2.865F-10         •0       2.8841E-10	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	6E-01       3.395E       02         6E-01       3.260E       02         6E-01       3.114E       02         6E-01       2.755E       02         6E-01       2.755E       02         6E-01       2.755E       02         6E-01       1.671E       02         6E-01       1.013E       02         6E-01       2.374E       00         4E-01       2.944E-01       02         2E-01       4.797E-09       03E-01         03E-01       0.0       02         2E-01       0.0       03E-02         9E-02       0.0       03E-02         9E-03       0.0       03E-02         9E-05       0.0       03E-01         9E-05       0.0       05E-11	L.J15E 03 4.962E 02 2.426E 02 4.057E 01 1.134E 00 2.478E-05 3.171E-02 2.478E-05 1.234E-28 3.598E-44 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	.902E 02 .265E 02 .732E 02 .184E 01 .948E 01 .948E 01 .948E 01 .948E 01 .948E 01 .948E 01 .948E 01 .052E-02 .05 .0 .0 .0 .0 .0 .0 .0 .0 .0 .0	7.634E 01 7.648E 01 7.4648 01 7.258E 01 6.861E 01 6.487E 01 2.498E 01 1.425E 01 1.425E 01 1.425E 01 1.425E 01 3.732E-C6 3.221E-23 0.0 0.0 0.0 0.0 0.0

Appendix A.2. Thermal Power of PWR Spent Fuel, High-Level Waste, and Structural Material Waste





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Thermal power (W/MTIHM)										
Time - (years)	TOTAL	PU239	AM241 PU2	0 CS137	SR 90	>R144	RH106	PU238	CS134 NB 95	ZR 95
$\begin{array}{c} 1.000 E-01\\ 3.000 E-01\\ 5.000 E-01\\ 1.000 E&00\\ 2.000 E&00\\ 3.000 E&00\\ 5.000 E&00\\ 1.000 E&01\\ 2.000 E&01\\ 3.000 E&01\\ 3.000 E&02\\ 1.000 E&02\\ 3.000 E&02\\ 1.000 E&02\\ 3.000 E&02\\$	4.778E 04 2.631E 04 1.945E 04 1.346E 04 5.389E 03 1.822E 03 1.822E 03 1.132E 03 8.644E 02 7.190E 02 2.843E 02 1.261E 02 5.474E 01	9.647E 00 9.647E 00 9.647E 00 9.647E 00 9.647E 00 9.647E 00 9.647E 00 9.647E 00 9.647E 00 9.642E 00 9.641E 00 9.641E 00 9.641E 00 9.569E 00	4.072E 00 1.6 5.400E 00 1.6 6.716E 00 1.6 9.947E 00 1.6 1.617E 01 1.6 3.309E 01 1.6 5.619E 01 1.6 5.619E 01 1.6 1.067E 02 1.6 1.246E 02 1.6 9.130E 01 1.5 2.971E 01 1.4	38E       01       5.015E         38E       01       4.992E         38E       01       4.969E         38E       01       4.969E         38E       01       4.969E         38E       01       4.969E         39E       01       4.969E         39E       01       4.969E         39E       01       4.969E         39E       01       4.969E         34E       01       3.969E         42E       01       3.167E         33E       01       4.9926E         99E       01       4.910E         99E       01       4.643E	02 4.890E 02 02 4.867E 02 02 4.845E 02 02 4.8788E 02 02 4.675E 02 02 4.675E 02 02 4.353E 02 02 3.865E 02 02 3.865E 02 02 3.045E 02 02 3.045E 02 02 3.045E 02 02 3.884E-01 01 3.884E-01 08 2.256E-08	8.471E 93 7.390E 03 5.932E 03 5.932E 03 1.5560E 03 1.5560E 03 1.378E 02 1.2552E 00 1.701E-04 2.306E-08 0.0 0.0	5.212E 03 4.542E 03 3.957E 03 2.806E 03 1.411E 03 7.095E 02 1.793E 02 5.760E 00 6.107E-03 6.638E-03 8.263E-27 0.0	7.409E 31 7.548E 01 7.645E 01 7.780E 01 7.780E 01 7.780E 01 7.673E 01 7.673E 01 6.817E 01 6.817E 01 3.628E 01 7.5502E 00 3.2055-02	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	$\begin{array}{cccccccccccccccccccccccccccccccccccc$
3.000E 03 1.000E 04 3.000E 04 1.000E 05 3.000E 05 1.000E 06	2.278E 01 1.352E 01 5.205E 00 1.053E 00 5.853E-01 3.907E-01	8.887E 00 7.318E 00 4.150E 00 5.545E-01 1.760E-03 1.106E-09	1.203E 00 1.2 3.216E-04 5.7 5.981E-05 6.8 1.983E-07 4.1 1.718E-14 1.7 0.0 1.7	DIE 01 3.960E- 19E 00 0.0 59E-01 0.0 22E-04 0.0 79E-08 0.0 59E-08 0.0	28 4.777E-29 0.0 0.0 0.0 0.0 0.0 0.0	0.0 0.0 0.0 0.0 0.0 0.0 0.0	0 • 0 0 • 0 0 • 0 0 • 0 0 • 0 0 • 0	2.852E-07 3.857E-21 0.0 0.0 0.0 0.0 0.0	0.0     0.0       0.0     0.0       0.0     0.0       0.0     0.0       0.0     0.0       0.0     0.0       0.0     0.0	5.5 5.0 5.0 0.0 0.0 0.0

Table A.4. Thermal power of PWR spent fuel as a function of decay time

Thermal power (W/MTIHM)

Time 🗌												
(years)	P0213	AT 217	FR221	NP237	P0214	LA140	AC225	TH229	U234	U233	P0218	RN222
1.000E-01 3.000E-01 5.000E-01 5.000E 00 3.000E 00 3.000E 00 1.000E 01 2.000E 01 1.000E 01 1.000E 02 3.000E 03 3.000E 03	4.585E-09 2.594E-09 2.594E-09 2.557E-09 2.629E-09 2.629E-09 2.891E-09 3.461E-09 5.084E-09 7.363E-09 4.356E-08 4.005E-08 7.474E-05 8.107E-04	3.953E-09. 2.236E-09 2.169E-09 2.204E-09 2.265E-09 2.336E-09 2.336E-09 2.492E-09 2.492E-09 2.492E-09 3.352E-09 3.755E-08 3.450E-07 5.363E-07 6.440E-05 5.766E-04	3.573E-09 2.022E-09 1.992E-09 2.042E-09 2.049E-09 2.049E-09 2.254E-09 2.254E-09 2.639E-09 3.963E-09 3.3963E-08 3.120E-08 3.120E-08 5.740E-09 5.740E-09 5.740E-09 5.740E-09 5.740E-09 5.740E-09 5.740E-09	9.535E-03 9.541E-03 9.541E-03 9.541E-03 9.546E-03 9.552E-03 9.552E-03 9.637E-03 9.637E-03 9.535E-02 1.913E-02 1.913E-02 3.584E-02 3.584E-02	7.411E-10 7.9555E-10 9.000E-10 1.207E-09 2.951E-09 5.522E-09 1.561E-08 5.196E-08 1.107E-07 1.216E-06 1.213E-06 1.420E-04 1.053E-03	4.631E 03 8.835E 01 1.685E 00 8.475E-05 2.143E-13 5.420E-22 3.467E-39 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	3.235E -09 1.830E-09 1.863E-09 1.863E-09 1.854E-09 1.854E-09 2.040E-09 2.040E-09 2.040E-09 3.586E-09 3.586E-09 3.073E-08 2.824E-07 4.390E-09 5.196E-096	1+229 1 • 541E - 09 1 • 549E - 09 1 • 557E - 09 1 • 557E - 09 1 • 674E - 09 2 • 138E - 09 3 • 142E - 09 3 • 142E - 09 4 • 691E - 08 2 • 475E - 06 4 • 618E - 06	0234 3.235E-02 3.240E-02 3.242E-02 3.253E-02 3.275E-02 3.229E-02 3.422E-02 3.422E-02 3.422E-02 3.595E-02 3.758E-02 4.592E-02 5.418E-02 5.418E-02 5.686E-02	U233 4.007E-07 4.085E-07 4.065E-07 4.065E-07 4.065E-07 5.242E-07 5.242E-07 5.078E-07 1.212E-06 1.628E-06 4.949E-06 1.833E-05 3.779E-05 3.779E-07	PU218 5.326E-10 6.172E-10 7.023E-10 9.429E-10 9.429E-10 1.544E-09 2.304E-09 1.219E-08 4.957E-08 8.664E-07 9.491E-07 9.472E-04 8.217E-04 8.217E-04	RN222 4.871E-13 5.6421E-10 8.620E-10 1.411E-09 2.107E-09 3.942E-08 3.713E-08 3.713E-08 3.713E-08 3.679E-07 8.661E-04 7.515E-04
1.000E 04 1.000E 05	8.193E-04 4.748E-03 1.851E-02	7.060E-04 4.091E-03 1.596E-02	6.386E-04 3.701E-03 1.443E-02	3.599E-02 3.576E-02 3.495E-02	6.078E-03 2.116E-02 4.822E-02 5.467E-02	0.0	5.779E-04 3.350E-03 1.306E-02	5.062E-04 2.934E-03 1.144E-02	5.593E-02 5.335E-02 4.540E-02	1.400E-03 4.139E-03 1.191E-02	4.746E-03 1.652E-02 3.764E-02	4.340E-03 1.510E-02 3.441E-92
1.000E 06	4.4996-02	3.877E-02	3.506E-02	2.611E-02	2.137E-02	0.0	3.175E-02	2.780E-02	1.197E-02	2.504E-02 2.635E-02	1.559E-J2	1.526E-02



Fig. A.5. Thermal power of PWR high-level waste as a function of decay time.

Thermal power (W/MTIHM)												
Time (years)	TOTAL	CS137	SR 90	AM243	AM241	PR144	RH106	PU239	NP237	CS134	20213	PU240
$\begin{array}{c} 1.000 = -01\\ 3.000 = -01\\ 1.000 = 00\\ 2.000 = 00\\ 3.000 = 00\\ 5.000 = 00\\ 1.000 = 01\\ 2.000 = 01\\ 2.000 = 01\\ 2.000 = 02\\ 1.000 = 02\\ 3.000 = 03\\ 3.000 = 03\\ 1.000 = 0$	$\begin{array}{c} 1.710 \pm 04\\ 1.323 \pm 94\\ 1.080 \pm 04\\ 7.325 \pm 03\\ 2.600 \pm 03\\ 1.486 \pm 03\\ 2.600 \pm 03\\ 1.486 \pm 03\\ 9.257 \pm 02\\ 6.707 \pm 02\\ 5.218 \pm 02\\ 6.495 \pm 00\\ 2.240 \pm 02\\ 1.941 \pm 02\\ 6.495 \pm 00\\ 7.739 \pm 00\\ 7.739 \pm 00\\ 7.739 \pm 00\\ 7.394 \pm 02\\ 7.394 \pm 02\\$	4.963E 02 4.941E 02 4.91F 02 4.91F 02 4.750E 02 4.641E 02 4.432E 32 3.948E 02 4.432E 32 3.948E 02 4.935E 01 4.862E-08 3.920E-28 0.0 0.0 0.0 0.0	4.639E 02 4.616E 02 4.736E 02 4.625E 02 4.516E 02 4.516E 02 3.623E 02 3.625E	5.485E-01 5.485E-01 5.485E-01 5.485E-01 5.484E-01 5.484E-01 5.484E-01 5.482E-01 5.430E-01 5.430E-01 5.430E-01 4.933E-01 4.938E-01 4.938E-01 4.938E-01 4.938E-01 4.938E-01 1.1	6.31CE 03 6.314E 00 6.319E 03 6.330E 00 6.350E 00 6.4636E 00 6.463E 03 6.466E 00 6.463E 03 6.464E 00 6.523E 03 6.518E 00 5.959E 00 4.335E 00 5.959E 00 4.335E 00 5.959E 00 1.411E 00 5.979E-05 1.985E-07 1.723E-14 0.0	5.733E 03 4.798E 03 2.572E 03 1.056E 03 4.332E 02 7.296E 01 1.151E-04 1.560E-08 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	3.854E 03 3.359E 03 2.928E 03 2.928E 03 2.928E 03 3.247E 02 1.326E 02 4.260E	4.823E-02 4.823E-02 4.824E-02 4.825E-02 4.825E-02 4.835E-02 4.835E-02 4.845E-02 4.845E-02 4.845E-02 4.8472E-02 5.280E-02 5.280E-02 1.2007E-01 1.047E-01 1.047E-01 1.03E-09 1.103E-09	9.538E-03 9.538E-03 9.539E-03 9.543E-03 9.543E-03 9.547E-03 9.5576E-03 9.5576E-03 9.5576E-03 9.5576E-03 1.303E-02 1.052E-02 1.052E-02 1.0572E-02 1.0573E-02 1.073E-02 1.073E-02 1.049E-03 7.840E-03	1.328E G3 1.241E G3 9.811E 02 7.010E 02 5.009E 02 2.557E 02 4.762E 01 1.651E 03 5.727E 02 4.764E-12 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.	2.518E-09 2.516E-09 2.516E-09 2.529E-09 2.529E-09 2.529E-09 2.529E-09 2.529E-09 3.836E-09 3.836E-09 3.836E-09 3.836E-09 3.477E-08 2.933E-07 3.2559E-03 1.4551E-03 1.222E-02 1.351E-02	8.236E-02 8.334E-02 8.431E-02 9.138E-02 9.587E-02 1.043E-01 1.229E-01 1.508E-01 2.065E-01 2.053E-01 1.539E-01 1.539E-01 1.539E-01 1.539E-01 1.529E-01 1.529E-01 1.529E-01 1.529E-01 1.529E-01 1.529E-01 1.529E-01 1.529E-01 1.529E-01 1.529E-01 1.529E-01 1.529E-01 1.529E-01
Time - (years)	AT217	FR221	AC225	TH229	U233	NB 95	SB126M	TC 99	CE144	ZR 95	EU154	CM244
$\begin{array}{c} 1.3000\pm-01\\ 3.000\pm-01\\ 1.000\pm03\\ 2.000\pm00\\ 3.000\pm00\\ 5.000\pm00\\ 1.000\pm01\\ 3.000\pm01\\ 3.000\pm01\\ 3.000\pm01\\ 3.000\pm01\\ 3.000\pm03\\ 1.000\pm03\\ 3.3000\pm03\\ 1.000\pm03\\ 3.3000\pm03\\ 1.000\pm03\\ 3.000\pm03\\ 1.000\pm03\\ 3.000\pm03\\ 1.000\pm03\\ 0.00\pm03\\ 1.000\pm03\\ 0.00\pm03\\ 0.00\pm$	$2 \cdot 170E - 09$ $2 \cdot 168E - 09$ $2 \cdot 168E - 09$ $2 \cdot 172E - 09$ $2 \cdot 180E - 09$ $2 \cdot 242E - 09$ $2 \cdot 242E - 09$ $2 \cdot 2459E - 09$ $3 \cdot 306E - 09$ $4 \cdot 704E - 09$ $2 \cdot 957E - 08$ $2 \cdot 957E - 08$ $1 \cdot 164E - 02$	1.963E-09 1.961E-09 1.961E-09 1.964E-09 1.964E-09 2.028E-09 2.028E-09 2.224E-09 2.224E-09 2.234E-07 2.234E-07 2.234E-07 2.234E-07 2.234E-03 2.220E-05 2.052E-04 1.131E-03 4.348E-03 3.9.528E-03 1.053E-02	1.776E-09 1.775E-09 1.775E-09 1.778E-09 1.778E-09 1.784E-09 1.836E-09 2.013E-09 2.706E-09 3.851E-09 2.453E-08 2.067E-07 2.299E-05 1.857E-03 3.935E-03 3.935E-03 3.528E-03	$1 \cdot 554E - 09 \\ 1 \cdot 555E - 09 \\ 1 \cdot 555E - 09 \\ 1 \cdot 553E - 09 \\ 1 \cdot 553E - 09 \\ 1 \cdot 573E - 09 \\ 1 \cdot 573E - 09 \\ 1 \cdot 573E - 09 \\ 2 \cdot 373E - 09 \\ 2 \cdot 373E - 09 \\ 2 \cdot 373E - 09 \\ 3 \cdot 373E - 05 \\ 1 \cdot 53E - 03 \\ 3 \cdot 44E - 04 \\ 3 \cdot 44E - 03 \\ 3 \cdot 44E - 03 \\ 3 \cdot 552E - 03 \\ 5 \cdot 552E - 03 \\ $	$\begin{array}{c} 6 \cdot 0.34 \pm -0.9\\ 1 \cdot 3.97 \pm -0.9\\ 2 \cdot 1.90 \pm -0.9\\ 4 \cdot 1.74 \pm -0.9\\ 8 \cdot 5 \pm 5 \pm -0.8\\ 1 \cdot 2.96 \pm -0.7\\ 2 \cdot 1.32 \pm -0.7\\ 4 \cdot 1.18 \pm -0.7\\ 1 \cdot 2.28 \pm -0.7\\ 1 \cdot 2.28 \pm -0.7\\ 1 \cdot 2.24 \pm -0.5\\ 1 \cdot 3.20 \pm -0.4\\ 1 \cdot 3.71 \pm -0.4\\ 1 \cdot 2.58 7 \pm -0.3\\ 3 \cdot 5.87 \pm -0.3\\ 3 \cdot 5.87 \pm -0.3\\ 7 \cdot 1.04 \pm -0.3\\ 7 \cdot 9.13 \pm -0.3\\ \end{array}$	1.889E 03 8.992E 02 4.177E 02 5.899E 01 1.173E 00 2.247E-02 7.930E-06 2.027E-14 1.324E-31 8.653E-49 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.	$\begin{array}{c} 9.895 \pm -03\\ 9.894 \pm -03$	$\begin{array}{c} 6.554 \pm -03\\ 6.553 \pm -03\\ 6.547 \pm -03\\ 6.547 \pm -03\\ 6.449 \pm -03\\ 6.344 \pm -03\\ 4.733 \pm -03\\ 2.463 \pm -04\\ 2.53 \pm -04\\ 4.53 \pm -04\\ 4.53$	5.173E 02 4.329E 02 3.623E 02 2.321E 02 9.525E 01 3.909E 01 6.584E 03 7.664E-09 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	9.933E 02 2.040502E 02 2.0401E 01 5.394E-01 1.0394E-01 1.0394E-03 3.772E-06 6.299E-32 4.116E-49 0.0 0.0 0.0 0.0 0.0 0.0 0.0	$\begin{array}{c} 8.94 \\ 0 \\ 0 \\ 0 \\ 0 \\ 0 \\ 0 \\ 0 \\ 0 \\ 0 \\ $	$\begin{array}{c} 5.227E & 01\\ 5.187E & 01\\ 5.147E & 01\\ 5.353E & 01\\ 4.863E & 01\\ 4.333E & 01\\ 3.578E & 01\\ 3.578E & 01\\ 1.664E & 01\\ 1.664E & 01\\ 1.142E & 00\\ 5.439E & 01\\ 1.28E & 14\\ 1.005E & 14\\$

#### Table A.5. Thermal power of PWR high-level waste as a function of decay time

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Fig. A.6. Thermal power of PWR structural material waste as a function of decay time.

Thermal power (W/MTIHM)												
Time (years) T	OTAL	CO 60	NI 59	NB 94	NI 63	AM241	NB 95	PU243	PU239	ZR 95	P0213	FE 55
1.000E-01 2 3.000E-01 1 5.000E-01 1 1.000E 00 1 1.000E 00 9 3.000E 00 8 5.000E 00 8 5.000E 01 3 2.000E 01 3 2.000E 01 2 1.000E 02 3 3.000E 02 3 1.000E 03 7 3.000E 04 3 1.000E 05 1 3.000E 05 2	-268E 02 -720E 02 -437E 02 -664E 01 -331E 01 -286E 01 -286E 01 -203E 01 -901E 00 -115E-01 -310E-02 -54E-02 -485E-02 -485E-02 -485E-02	1.115E 02 1.086E 02 1.058E 02 9.906E 01 8.685E 01 7.615E 01 5.853E 01 3.032E 01 8.138E 00 2.184E 00 2.184E 00 2.184E 00 2.184E 00 0.191E-04 8.236E-16 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.	3.299E-02 3.299E-02 3.299E-02 3.299E-02 3.299E-02 3.298E-02 3.298E-02 3.298E-02 3.298E-02 3.298E-02 3.298E-02 3.298E-02 3.298E-02 3.296E-02 3.270E-02 3.298E-02 3.2178E-02	$\begin{array}{c} 1 & \cdot 307E - 02 \\ 1 & \cdot 306E - 02 \\ 1 & \cdot 294E - 03 \\ 4 & \cdot 634E - 04 \\ 4 & \cdot 633E - 07 \\ \end{array}$	$\begin{array}{c} 2.613E-01\\ 2.609E-01\\ 2.509E-01\\ 2.576E-01\\ 2.5576E-01\\ 2.5576E-01\\ 2.5576E-01\\ 2.520E-01\\ 2.2086E-01\\ 1.231E-02\\ 2.23E-02\\ 1.397E-04\\ 3.9990E-11\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0$	3.483E-03 4.133E-03 6.358E-03 9.405E-03 1.230E-02 1.768E-02 2.898E-02 4.450E-02 4.450E-02 4.450E-02 4.562E-02 4.562E-02 1.485E-02 6.011E-04 1.609E-07 2.991E-08 9.917E-11 8.599E-18	5.571E 01 2.651E 01 1.231E 01 1.739E 00 3.459E-02 2.337E-07 5.9075E-16 3.9075E-16 3.9075E-50 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	6.191E-03 8.192E-03 8.192E-03 8.193E-03 8.194E-03 8.203E-03 8.203E-03 8.203E-03 8.209E-03 8.209E-03 7.997E-03 7.997E-03 7.997E-03 7.997E-03 3.459E-03 3.459E-03 3.459E-03 3.459E-03 3.8	4.925E-03 4.825E-03 4.825E-03 4.825E-03 4.825E-03 4.825E-03 4.825E-03 4.825E-03 4.825E-03 4.824E-03 4.824E-03 4.824E-03 4.824E-03 4.696E-03 3.669E-03 2.076E-03 2.076E-04 8.789E-07	$\begin{array}{c} 2.928E & 01\\ 1.327E & 01\\ 6.014E & 00\\ 8.315E-01\\ 1.590E-02\\ 3.041E-07\\ 2.842E-16\\ 1.857F-33\\ 1.213E-50\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0$	1.261E-12 1.265E-12 1.272E-12 1.331E-12 1.373E-12 1.467E-12 1.467E-12 1.455E-12 2.196E-12 2.196E-10 3.116E-08 4.098E-07 2.374E-06 2.035E-05	6.679E 03 6.332E 00 5.254E 00 3.083E 00 1.809E 03 4.777E-01 2.306E-03 1.812E-11 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0

Table A.6.	Thermal power	of PWR	structural	material	waste	as	a	function	of	decay	time
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Thermal power (W/MTIHM)

Time												
(voare)	*****		~~ ~^	60001	AC225	ND 074	THOOD	50105	10.077	Due 70	11223	00314
(Jears)	CS137	A1217	5R 90	FR221	ALZZO	NR ADW	18229	28152	NP 237	PU238	0233	PU214
				0 0045 13	0 0075 17		3 30/5 13					
1.000E-01	2.483E-01	1+0805-15	2+421E-01	9+820E-13	8.893E-13	1.745E-00	(. (9 DE-13	4.424E 00	4.771E-06	3-831E-02	2.090E-10	4-5596-13
3.000E-01	2.4716-01	1.090E-12	2.409E-01	9.862E-13	8.926E-13	1.944E-06	7.837E-13	4.208E 00	4.772E-05	3-863E-02	2.130E-10	5.200E-13
5.000E-01	2.460E-01	1.096E-12	2.398E-01	9.915E-13	8.974E-13	2.1418-06	7.880E-13	4.003E 00	4.772E-06	3.885E-02	2.170E-10	5.834E~13
1.000E 00	2.432E-01	1.115E-12	2.369E-01	1.009E-12	9.128E-13	2.624E-06	7.990E-13	3.532E 00	4.773E-06	3.911E-02	2.269E-10	7.598E~13
2.000E 00	2.376E-01	1.148E-12	2,313E-01	1.038E-12	9.394E-13	3.555E-06	8.227E-13	2.750E 00	4.775E-06	3.908E-02	2.489E-10	1.189E-12
3.000F 00	2.322E-01	1.184F-12	2-259F-01	1.070E-12	9-689E-13	4-439E-06	8-485E-13	2.141E 00	4.778E-06	3.883E-02	2.739E-10	1.722E-12
5-000F 00	2.2175-01	1.265F-12	2-154F=01	1-144E-12	1-035E-12	6-078E-06	9-066E-13	1.298F 00	4-787E-06	3-824F-02	3-128E-10	3-098E-12
1.0005 01	1 0755-01	1.5165-12	1 0125-01	1.7715-12	1.2415-12	0.5145-06	1.0875-12	3.7145-01	A. 822E-06	3.676E-02	4.126E-10	8.3766-12
I OUVE VI	1.9/55-01	1.5105-15	1.9120-01		1 0045-10	1 1015-00	1 5075-12	3 0415-01	4 022L-00	3 3075-02	6 1675-10	2 70 36-11
2.000E 01	1.508E-01	2.228E-12	1.507E-01	2.0155-12	1.0245-12	1.4245-05	1.024/5-15	3.0412-02	4.9342-00	3.3976-02	0.1332-10	E 603E-11
3.000E 01	1.244E-01	3.223E-12	1.188E-01	2.915E-12	2.039E-12	1.7082-05	2.311E-12	2.490E-03	5. JUIE-00	3.139E-02	8.2345-10	2.040E-11
1.000E 02	2.468E-02	1.893E-11	2.245E-02	1.7128-11	1.549E-11	2.123E-05	1 • 35 7E-11	6.146E-11	6.370E-06	1.808E-02	2.486E-09	6.135E-13
3.000E 02	2.431E-04	1.731E-10	1.924E-04	1.566E-10	1.417E-10	2.135E-05	1.241E-10	0.0	9.5758-06	3.7398-03	9.184E-09	6.085E~09
1.000E 03	2.298E-11	2.685E-09	1.118E-11	2.429E-09	2.198E-09	2.134E-05	1.925E-09	0.0	1.529E-05	1.599E-05	4.683E-08	7.106E-08
3.000F 03	1.960E-31	3-222F-08	2.367F-32	2-914E-08	2.638F-08	2-132E-05	2.310E-08	0.0	1.793E-35	1.423E-10	1.891E-07	5.265E-07
1.0005 04		3. 6325-07	0.0	3.19AF-07	2.891F-07	2.126F-05	2-532F-07	0.0	1.800F-05	1.925E-24	7-001E-07	3-0435-36
3 0000 04		2 0465-06	<b>~~~~~</b>	1 4615-06	1.6755-06	2.1065-06	1.4675-06	0.0	1.788F-05	0.0	2-0705-06	1-058E-05
3.0000 04	4.4	2.0402-00	<b>V</b> •V	1.0512-00	1.07 32-00	2.1000-05	5 7015-06		1 7405-05		E 0E75-06	2 4115-05
1.300E 03	0.0	1.980E-06	0.0	1.218E-06	0.533E-00	2+0412-05	5.721E-00	0.0	1.7402-05	0.0	5.957E-00	2.4112-03
3.000E 05	0.0	1.754E-05	0.0	1.586E-05	1.436E-05	1.864E-05	1.257E-05	0.0	1.0395-05	0.0	1.183E-03	2.7345-95
1.000E 06	0.0	1.939E-05	0.0	1.754E-05	1.587E-05	1.357E-05	1.390E-05	0.0	1.306E-05	0.0	1.318E-05	1.069E-05

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Appendix A.3. Toxicity of PWR Spent Fuel, High-Level Waste, and Structural Material Waste

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Fig. A.7. Toxicity of PWR spent fuel as a function of decay time.

	Toxicity (m <sup>3</sup> water/MTIHM)											
Time (years)	TOTAL	SR 90	RA226	PR143	PU239	PU240	AM241	P8210	CE 1'44	TH229	RJ106	RA225
1.000E-01 3.000E-01 1.000E 00 2.000E 00 3.000E 00 5.000E 01 3.000E 01 3.000E 01 3.000E 01 1.000E 01 1.000E 03 1.000E 03 1.000E 04 1.000E 05 3.000E 05 3.000E 05 1.000E 06	5.160E 13 1.732E 12 4.605E 11 3.482E 11 2.605E 11 2.605E 11 2.018E 11 1.508E 11 1.508E 11 1.508E 11 1.508E 11 1.477E 10 1.107E 09 3.866E 08 9.431E 07 5.505E 07 5.485E 07 5.955E 07 2.713E 07	2.468E 11 2.457E 11 2.4457E 11 2.4457E 11 2.360E 11 2.360E 11 1.537E 11 1.537E 11 1.537E 11 1.2212E 11 2.412E 10 1.960E 08 1.138E 08 1.412E-20 0.0 0.0 0.0 0.0	$\begin{array}{c} 4.952E-01\\ 5.678E-01\\ 6.460E-01\\ 8.672E-01\\ 1.420E 00\\ 2.120E 00\\ 3.966E 00\\ 1.121E 01\\ 3.732E 01\\ 7.951E 02\\ 8.714E 03\\ 1.0258E 05\\ 4.367E 06\\ 1.520E 07\\ 3.463E 07\\ 3.927E 07\\ 1.535E 07\\ \end{array}$	$\begin{array}{c} 5.069E & 13\\ 1.213E & 12\\ 2.902E & 10\\ 2.570E & 06\\ 2.014E - 02\\ 1.580E - 21\\ 0.080E - 22\\ 0.080E - 22$	$\begin{array}{c} \textbf{ . 260E } 07\\ \textbf{ 6 . 250E } 07\\ \textbf{ 6 . 260E } 07\\ \textbf{ 6 . 256E } 07\\ \textbf{ 6 . 245E } 07\\ \textbf{ 6 . 2945E } 07\\ \textbf{ 6 . 2945E } 07\\ \textbf{ 6 . 2945E } 07\\ \textbf{ 3 . 597E } 06\\ \textbf{ 1 . 142E } 06\\ \textbf{ 1 . 142E } 06\\ \textbf{ 7 . 177E } 03\\ \end{array}$	$\begin{array}{c} 1.052E 08\\ 1.052E 08\\ 1.052E 08\\ 1.052E 08\\ 1.053E 08\\ 1.053E 08\\ 1.053E 08\\ 1.053E 08\\ 1.054E 08\\ 1.054E 08\\ 1.054E 08\\ 1.054E 08\\ 1.0454E 08\\$	$\begin{array}{c} 3.064E 07\\ 4.065E 07\\ 5.053E 07\\ 7.487E 07\\ 1.217E 08\\ 1.663E 08\\ 2.490E 08\\ 4.228E 08\\ 6.618E 08\\ 4.228E 08\\ 6.618E 08\\ 9.381E 08\\ 6.872E 08\\ 9.381E 08\\ 6.872E 08\\ 2.36E 08\\ 2.421E 08\\ 1.602E 06\\ 1.492E 00\\ 1.492E 00\\ 1.492E 00\\ 1.293E-07\\ 0.0 \end{array}$	5.092E-03 6.072E-03 7.158E-03 1.052E-02 3.602E-02 3.929E-01 2.180E-02 3.929E-01 2.180E-00 6.252E-00 1.440E-02 2.092E-03 3.058E-02 1.309E-02 1.440E-02 2.092E-03 1.309E-04 0.160E-02 1.440E-02 2.092E-03 1.440E-02 2.092E-03 1.440E-02 2.092E-03 1.440E-02 1.460E-02 1.440E-02 1.460E-02 1.4	$\begin{array}{c} 1 \cdot 153E \ 11\\ 9 \cdot 645E \ 10\\ 8 \cdot 072E \ 10\\ 5 \cdot 170E \ 10\\ 2 \cdot 122E \ 10\\ 8 \cdot 709E \ 09\\ 1 \cdot 709E \ 00\\ 0 \cdot 0\\ 0 \cdot $	$\begin{array}{c} 1 & 259 \\ \hline 1 & 265 \\ \hline -01 \\ 1 & 273 \\ \hline -01 \\ 1 & 327 \\ \hline -01 \\ 1 & 368 \\ \hline -01 \\ \hline -01$	5.433E 10 4.735E 10 2.925E 10 2.925E 10 7.396E 09 1.869E 09 6.005E 09 6.005E 07 6.369E 01 8.614E-20 0.0	$\begin{array}{c} 1.402E-0.1\\ 1.024E-01\\ 1.017E-01\\ 1.032E-01\\ 1.0562E-01\\ 1.168E-01\\ 2.054E-01\\ 2.054E-01\\ 2.975E-01\\ 2.975E-01\\ 1.760E\\ 00\\ 1.617E\\ 01\\ 2.514E\\ 02\\ 3.019E\\ 03\\ 3.309E\\ 1.918E\\ 05\\ 7.478E\\ 05\\ 1.664E\\ J.61\\ 1.817E\\ 06\\ 00\\ 1.817E\\ 00\\ 00\\ 00\\ 00\\ 00\\ 00\\ 00\\ 00\\ 00\\ 0$

Table A.7. Toxicity of PWR spent fuel as a function of decay time

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Toxicity (m<sup>3</sup> water/MTIHM)

Time (years) <b>CS137</b>	P0 21 0	RN219	CS134	1129	AM243	NP237	TH230	PU238	PU242	SR 89	AC225
$\begin{array}{c} 1.000E-01 & 5.2031\\ 3.000E-01 & 5.1739\\ 5.000E-01 & 5.1551\\ 1.000E & 00 & 5.0951\\ 2.000E & 00 & 4.9801\\ 3.000E & 00 & 4.8651\\ 5.000E & 01 & 4.6461\\ 1.000E & 01 & 4.6461\\ 3.000E & 01 & 2.6077\\ 1.000E & 02 & 5.0931\\ 1.000E & 02 & 5.0931\\ 1.000E & 03 & 4.8177\\ 3.000E & 03 & 4.8177\\ 1.000E & 03 & 4.8177\\ 1.000E & 03 & 4.8177\\ 1.000E & 03 & 4.007\\ 1.000E & 05 & 0.03\\ 3.000E & 05 & 0.0\\ 3.000E & 05 & 0.0\\ 1.000E & 06 & 0.0\\ \end{array}$	09 4.206E-0 09 5.329E-0 09 6.555E-0 09 2.131E-0 09 3.901E-0 09 3.901E-0 09 3.901E-0 09 5.613E-0 09 5.613E-0 09 8.932E-0 08 2.958E 0 06 2.988E 0 -21 3.237E 0 6.510E 0 1.4830E 0 1.483E 0 6.577E 0	9.710E 00 1.213E 01 1.469E 01 3.2.171E 01 3.3.550E 01 3.4.635E 01 2.1.375E 02 1.2.635E 02 1.2.635E 02 1.2.659E 03 3.4669E 03 3.4669E 03 3.4669E 03 5.1.90E 05 5.3.307E 05 5.3.307E 05 5.3.325E 05 9.3355E 05	$\begin{array}{c} 1.680E 10\\ 1.571E 10\\ 1.4691E 10\\ 1.249E 10\\ 8.870E 09\\ 3.235E 09\\ 6.338E 09\\ 3.235E 09\\ 6.324E 09\\ 2.090E 07\\ 7.246E 05\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ $	5.244E 055 5.261E 055 5.268E 055 5.2	4.267E 06 4.267E 06 4.267E 06 4.267E 06 4.267E 06 4.267E 06 4.267E 06 4.262E 06 4.262E 06 4.256E 06 4.256E 06 3.220E 06 3.220E 06 1.6668E 06 2.5534E 02 9.231E-02 9.231E-02 9.231E-03	1.040E 05 1.041E 05 1.041E 05 1.041E 05 1.041E 05 1.042 05 1.042 05 1.044E 05 1.044E 05 1.075E 05 1.075E 05 1.107E 05 1.335E 05 3.3924E 05 3.9924E 05 3.924E 05 3.812E 05 2.850E 05	1.204E 01 1.305E 01 1.407E 01 2.169E 01 2.633E 01 3.716E 01 1.6354E 01 1.184E 02 1.758E 02 2.245E 03 8.421E 02 2.245E 03 8.421E 04 2.592E 04 8.412E 04 2.55.168E 05 5.873E 05 2.301E 05	4.473E 08 4.555E 08 4.614E 08 4.694E 08 4.724E 08 4.631E 08 4.631E 08 4.631E 08 4.631E 08 4.631E 08 4.527E 07 1.934E 08 2.1895E 08 4.527E 07 1.934E 00 2.327E-14 0.0 0.0 0.0	3.515E 05 3.515E 05 3.513E 05 3.513E 05 3.513E 05 3.454E 05 3.454E 05 2.938E 05 2.054E 05 2.054E 05 2.054E 05	1.767E 11 5.484E 10 2.379E 10 1.940E 09 1.290E 07 8.575E 00 4.917E-11 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	$\begin{array}{c} 1.852E-02\\ 1.048E-02\\ 1.017E-02\\ 1.053E-02\\ 1.054E-02\\ 1.054E-02\\ 1.398E-02\\ 1.398E-02\\ 2.354E-02\\ 2.975E-02\\ 2.975E-02\\ 2.975E-02\\ 1.760E-01\\ 1.514E\\ 3.019E\\ 02\\ 3.019E\\ 02\\ 3.019E\\ 02\\ 3.019E\\ 04\\ 1.644E\\ 05\\ 1.817E\\ 05\\ \end{array}$

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Fig. A.8. Toxicity of PWR high-level waste as a function of decay time.

								Toxi	city (m <sup>3</sup> wat	er/MTIHM)					
Time (years)	TOTAL,		SR 90		AM243	AM241		TH229	RA226	RA225	CE144	PU239	PU240	RU106	98219
$\begin{array}{c} 1.000E-01\\ 3.000E-01\\ 5.000E-01\\ 2.000E 00\\ 3.000E 00\\ 5.000E 00\\ 1.000E 01\\ 3.000E 01\\ 3.000E 01\\ 3.000E 01\\ 1.000E 03\\ 1.000E 03\\ 1.000E 03\\ 1.000E 03\\ 1.000E 03\\ 1.000E 05\\ 3.000E 05\\ 1.000E 06\\ 0.000E 06\\ 0.00$	4.3 19E 3.777E 3.777E 3.137E 2.725E 2.506E 2.506E 2.3279E 1.556E 2.3279E 1.556E 2.3279E 1.556E 2.3279E 1.556E 2.3210E 3.795E 2.208E 2.577E 2.208E 2.577E 1.688E	111 111 111 111 111 110 0766666 066666	2.442E 2.430E 2.390E 2.333E 2.333E 2.173E 1.520E 1.1520E 1.125E 2.940E 1.127E 2.385E 0.0 0.0 0.0 0.0	11 11 11 11 11 11 11 11 11 10 08 01 -20	4.266E 00 4.266E 00 4.265E 00 4.265E 00 4.265E 00 4.265E 00 4.265E 00 4.265E 00 4.265E 00 4.265E 00 4.256E 00 4.265E 00 1.668E 00 3.265E 00 1.668E 00 3.265E 00 3.668E	4.749E 4.752E 4.754E 4.754E 4.779E 4.819E 4.8665 4.909E 4.909E 4.909E 4.4902E 4.4992E 4.4992E 1.4052E 1.4052E 1.297E 0.0	077 077 077 077 077 077 077 077 003 0007 -07	$\begin{array}{c} 1 \cdot 2 \ 7 \ 0 \ E \ - 0 \ 1 \\ 1 \cdot 2 \ 7 \ 0 \ E \ - 0 \ 1 \\ 2 \ 7 \ 0 \ E \ - 0 \ 1 \\ 2 \ 7 \ 2 \ - 0 \ 1 \\ 2 \ 7 \ 2 \ - 0 \ 1 \\ 2 \ 7 \ 2 \ 7 \ - 0 \ 1 \\ 2 \ 7 \ 7 \ - 0 \ 1 \\ 2 \ 7 \ 5 \ 6 \ - 0 \ 1 \\ 1 \cdot 3 \ 1 \ 4 \ E \ - 0 \ 1 \\ 2 \ 7 \ 5 \ 6 \ - 1 \ 7 \ 5 \ 6 \ - 1 \ 7 \ 2 \ 5 \\ 5 \ - 1 \ 7 \ 2 \ 5 \ 5 \ 5 \ 5 \ 5 \ 5 \ 5 \ 5 \ 5$	6.606E-01 7.399E-01 8.194E-00 1.018E00 1.814E00 2.612E00 8.716E00 1.294E01 4.622E01 2.531E02 2.476E05 1.613E04 1.046E05 8.627E05 8.607E05 1.845E05	$\begin{array}{c} 1.016E-01\\ 1.016E-01\\ 1.016E-01\\ 1.018E-01\\ 1.022E-01\\ 1.051E-01\\ 1.55E-01\\ 2.205E-01\\ 2.205E-01\\ 1.405E\\ 00\\ 1.184E\\ 01\\ 1.405E\\ 03\\ 1.663E\\ 04\\ 5.860E\\ 05\\ 4.937E\\ 05\\ 5.455E\\ 05\\ \end{array}$	7.799E 10 6.527E 10 5.469E 10 5.469E 10 1.436E 10 5.893E 10 1.436E 09 9.925E 08 1.155E 03 2.122E-01 0.0 0.0 0.0 0.0 0.0 0.0 0.0	3.130E 05 3.130E 05 3.130E 05 3.131E 05 3.131E 05 3.131E 05 3.135E 05 3.146E 05 3.146E 05 3.146E 05 3.250E 05 3.426E 05 3.426E 05 5.359E 05 7.788E 05 6.793E 05 1.023E 05 3.236E 02 7.156E-03	5.290E 05 5.353E 05 5.415E 05 5.570E 05 5.869E 05 6.158E 05 6.702E 05 7.894E 05 7.894E 05 1.090E 06 1.327E 06 1.327E 06 1.327E 06 1.327E 06 1.327E 06 5.647E 01 5.647E 01 5.945E-04 6.121E-04	$\begin{array}{c} 4.019 \pm 10\\ 3.552 \pm 10\\ 3.552 \pm 10\\ 1.58 \pm 10\\ 1.383 \pm 10\\ 1.383 \pm 09\\ 4.442 \pm 07\\ 4.560 \pm 04\\ 4.752 \pm 01\\ 5.915 \pm -20\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0$	$\begin{array}{c} \textbf{7.291E-03} \\ \textbf{8.540E-03} \\ \textbf{9.933E-03} \\ \textbf{1.401E-02} \\ \textbf{2.473E-02} \\ \textbf{3.877E-02} \\ \textbf{7.645E-02} \\ \textbf{2.225E-01} \\ \textbf{1.391E} \\ \textbf{0.0} \\ \textbf{9.673E} \\ \textbf{0.0} \\ \textbf{9.673E} \\ \textbf{0.1} \\ \textbf{7.426E} \\ \textbf{0.25} \\ \textbf{0.1} \\ \textbf{7.426E} \\ \textbf{0.25} \\ \textbf{0.1} \\ \textbf{7.426E} \\ \textbf{0.25} \\ \textbf{0.2581E} \\ \textbf{0.5534E} \\ \textbf{0.4} \\ \textbf{0.5534E} \\ \textbf{0.4} \\ \textbf{0.5534E} \\ \textbf{0.5554E} \\$
						*		Toxt	icity (m <sup>3</sup> wai	er/MTIHM)					
Time (years)	NP 237		SN 126		CS137	CS134		SE 79	PD107	SR 89	TC 99	AC225	PR143	NP239	P0210
$\begin{array}{c} 1.3000 \pm -01\\ 3.000 \pm -01\\ 5.000 \pm 00\\ 2.300 \pm 00\\ 3.000 \pm 00\\ 5.000 \pm 00\\ 1.000 \pm 01\\ 3.000 \pm 01\\ 3.000 \pm 01\\ 3.000 \pm 01\\ 3.000 \pm 02\\ 1.000 \pm 03\\ 1.000 \pm $	1.040E 1.040E 1.040E 1.041E 1.041E 1.041E 1.041E 1.041E 1.041E 1.041E 1.041E 1.041E 1.041E 1.05E 1.180E 1.180E 1.178E 1.178E 1.145E 1.073E 8.551E	<b>0000000000000000000000000000000000000</b>	2.590EE 2.590E	05555555555555555555555555555555555555	5.148E 0 5.124E 0 5.124E 0 5.042E 0 4.817E 0 4.8197E 0 4.8597E 0 4.8597E 0 4.8597E 0 4.8597E 0 2.5580E 0 5.119E 0 5.143E 0 5.043E 0 0.0 5.043E 0 0.0 0.0 0.0	1.4492   1.3552   1.25712   1.2712   7.54682   2.579282   2.579282   3.57712   3.57712   0.000   0.000   0.000   0.000   0.000   0.000	10 10 10 099 099 075 -05	$\begin{array}{c} 1.364 \pm 05\\ 1.364 \pm 05\\ 1.364 \pm 05\\ 1.354 \pm 05\\ 1.354 \pm 05\\ 1.354 \pm 05\\ 1.363 \pm 05\\ 1.363 \pm 05\\ 1.363 \pm 05\\ 1.363 \pm 05\\ 1.359 \pm 05\\ 1.359 \pm 05\\ 1.359 \pm 05\\ 1.3226 \pm 05\\ 1.3226 \pm 05\\ 1.3226 \pm 05\\ 1.552 \pm 05\\ 3.167 \pm 05\\ 3.167 \pm 05\\ 0.$	3.740E 04 3.740E 04 3.739E 04 3.739E 04 3.739E 04 3.720E 04 3.700E 04 3.622E 04 3.361E 04	1.965E 13 7.210E 09 2.2645E 03 2.2645E 03 2.257E 08 1.435E 06 9.535E 03 4.213E-01 5.469E-12 5.0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0		1.)17E-02 1.016E-02 1.016E-02 1.018E-02 1.022E-02 1.550E-02 2.255E-02 1.455E-02 1.455E-02 1.455E-02 1.455E-01 1.184E 00 1.155E 22 1.056E 03 2.253E 04 4.937E 04 5.455E 04	1.426E 10 3.411E 08 8.162E 06 7.228E 02 5.667E-06 4.444E-14 2.732E-30 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	1.707E 05 1.706E 05 1.706E 05 1.706E 05 1.706E 05 1.706E 05 1.705E 05 1.705E 05 1.703E 05 1.703E 05 1.703E 05 1.703E 05 1.659E 05 1.553E 05 1.287E 05 1.287E 05 1.287E 05 1.3553E 05 1.287E 05 1.3553E 05 1.287E 05 1.3553E 05 1.287E 05 1.3553E 05 1.287E 05 1.3553E 05 1.35552E 05 1.3555E 05 1.35555E 05 1.3555E 05 1.3555E 05 1.3555E 05 1.3555E 05 1.3555E	$\begin{array}{c} 6.794 \pm -04\\ 3.130 \pm -04\\ 9.632 \pm -04\\ 1.416 \pm -03\\ 2.574 \pm -03\\ 3.6574 \pm -03\\ 3.2574 \pm -03\\ 3.2574 \pm -03\\ 3.179 \pm -02\\ 1.007 \pm -01\\ 1.988 \pm $

Table A.8. Toxicity of PWR high-level waste as a function of decay time

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Fig. A.9. Toxicity of PWR structural material waste as a function of decay time.

	Toxicity (m <sup>3</sup> water/MTIHM)										
Time (years)	TOTAL	NB 94	SR 90 SN119	M R4226	CD 60	NI 63	NI 57	P9210	AM241 PU2	39 PU24 J	
1.000E-01 3.000E-01 5.000E-01 2.000E 00 3.000E 00 3.000E 00 1.000E 01 1.000E 01 3.000E 01 1.000E 02 1.000E 03 3.000E 03 1.000E 04 3.000E 04	2.056E 09 1.642E 09 9.280E 08 5.439E 08 3.877E 08 2.779E 08 1.883E 08 2.779E 08 1.158E 08 6.491E 07 3.303E 06 6.518E 05 3.764E 05 2.012E 05 5.256E 04	$\begin{array}{c} 4.275 \pm 05 \\ 4.274 \pm 05 \\ 4.274 \pm 05 \\ 4.274 \pm 05 \\ 4.261 \pm 05 \\ 4.261 \pm 05 \\ 4.132 \pm 05 \\ 3.039 \pm 05 \\ 3.039 \pm 05 \\ 1.535 \pm 05 \\ 1.406 \pm 04 \end{array}$	1.221E 08 1.267 1.216E 08 1.031 1.210E 08 8.383 1.196E 08 5.001 1.167E 08 1.779 1.140E 08 6.322 1.687E 08 8.018 9.651E 07 4.586 7.606E 07 1.493 5.996E 07 4.860 1.133E 07 0.0 9.713E 04 0.0 5.641E-03 0.0 1.195E-23 0.0 0.0 0.0 0.0	E 09 3.308E-04 E 09 3.734E-04 E 08 4.190E-04 E 08 5.457E-04 E 08 5.457E-04 E 06 2.225E-03 E 06 2.225E-03 E 04 $\pounds$ .015E-03 E 04 $\pounds$ .015E-03 E 04 $\pounds$ .015E-01 4.406E-01 4.406E-01 3.781E 02 2.183E 03 7.599E 03 1.732E 03	2.411E 06 2.348E 08 2.287E 08 2.142E 08 1.646E 08 1.265E 08 6.556E 07 1.760E 07 1.760E 07 1.760E 07 2.4.722E 06 4.736E 02 1.781E-09 0.0 0.0	2.193E 07 2.190E 07 2.179E 07 2.179E 07 2.162E 07 2.162E 07 2.114E 07 2.036E 07 1.888E 07 1.888E 07 1.751E 07 1.033E 07 1.033E 06 1.172E 04 3.349E-03 0.0 0.0	2.591E 04 2.591E 04 2.591E 04 2.591E 04 2.591E 04 2.591E 04 2.591E 04 2.590E 04 2.590E 04 2.590E 04 2.590E 04 2.590BE 04 2.588E 04 2.5568E 04 2.5568E 04 2.5568E 04 2.5376E 04 1.9998E 04	3.647E-06 4.275E-06 7.120E-06 1.326E-05 2.237E-05 2.173E-04 1.152E-03 3.247E-03 3.247E-03 3.247E-03 1.52E-03 3.247E-03 1.530E 01 1.530E 01 1.530E 02 2.547E 02 2.279E 03	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	
3.000E 05 1.000E 06	3.187E 04 1.362E 04	1.515E 01 6.308E-10	0.0 0.0 0.0	1.964E 04 7.677E 03	0.0	0.0	1.926E 03 4.484E 00	5.889E 03 2.302E 03	6.471E-11 5.6 9.0 3.5	98E D0 5.714E-05 90E-06 5.682E-05	

## Table A.9. Toxicity of PWR structural material waste as a function of decay time

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Toxicity (m<sup>3</sup> water/MTIHM)

Time (years)	TH229	RA225	NB 95	ZP 95	RN219	PD210	CE144	SN123	C\$137	SB125	RU1 36	NB 93M
(years) 1.000E-01 3.000E-01 1.000E 30 3.000E 30 3.000E 30 5.000E 01 3.000E 01 3.000E 01 3.000E 03 3.000E 03 3.000E 03 3.000E 04 1.000E 05 3.000E 05	TH229 6.370E-05 6.405E-05 6.529E-05 6.529E-05 6.934E-05 8.882E-05 1.305E-04 1.888E-04 1.109E-03 1.014E-02 1.573E-01 1.888E 00 2.069E 01 1.199E 02 4.675E 02 1.028E 03	RA225 5.092E-05 5.143E-05 5.244E-05 5.378E-05 5.378E-05 5.927E-05 7.105E-05 1.044E-04 8.870E-04 8.870E-04 8.870E-04 8.870E-04 1.5510E 00 1.6555E 01 9.590E 01 3.740E 02 8.22CE 02	NB 95 1.161E 08 5.526E 07 2.567E 07 3.625E 06 7.211E 03 4.873E-03 4.873E-03 8.138E-27 5.317E-44 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	ZP 95 9.633E 07 4.366E 07 1.979E 07 2.736E 06 5.231E 04 1.001E C3 3.658E-01 9.351E-10 6.109E-27 3.51E-10 6.109E-27 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.	RN219 7.624E-C3 8.875C-02 2.057F-02 2.057F-02 2.6598E-02 7.154E-02 7.154E-02 7.154E-02 1.345E-01 1.9699-C1 6.251E-01 1.830E 00 1.830E 01 1.654E 02 3.773E 02 3.773E 02 3.6765E 02	PD210 3.399E-07 4.067E-07 4.822E-07 7.132E-07 2.461E-06 6.060E-06 3.105E-05 1.646E-04 4.640E-04 1.500E-02 1.500E-02 1.505E 01 3.255E 02 7.418E 02 8.413E 02	CE144 3.902E 07 3.265E 07 2.732E 07 7.184E 06 2.948E 06 4.965E 05 5.782E 03 7.836E-01 1.062E-04 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	SN123 3.900E 07 2.635E 07 1.781E 07 6.682E 05 1.326E 05 1.326E 05 1.458E-01 4.482E-13 1.377E-18 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.	CS137 2.575E 05 2.554E 06 2.552E 06 2.465E 06 2.4049E 06 2.3049E 06 1.626E 06 1.626E 06 2.561E 05 2.521E 06 2.521E 06 2.521E 06 2.521E 06 2.522E 05 2.522E 05 2.522E 05 2.522E 05 2.522E 05 2.522E 06 2.522E 05 2.522E 06 2.522E 0	SB125 1.415E 07 1.346E 07 1.280E 07 1.130E 07 6.849E 06 4.152E 06 1.188E 06 4.152E 06 1.188E 06 4.152E 06 1.188E 06 4.152E 06 1.966E-04 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	RUI 36 2.011E 07 1.752E 07 1.527E 07 1.083E 07 5.444E 06 2.737E 06 6.918E 07 2.222E 01 2.313E 01 2.313E 01 2.313E 01 2.956E-23 0.0 0.3 0.3 0.3 0.3 0.3 0.3 0.	NE 9 3M 2.462E 01 2.743E 01 3.703E 01 5.016E 01 5.016E 01 1.54263E 01 1.542E 02 2.996E 02 2.996E 02 2.996E 02 2.996E 02 2.999E 02 2.999E 02 2.972E 02 2.972E 02 2.977E
1.000E 06	1.136E 03	9.089E 02	0.0	0.0	4.070E UZ	30209C UZ	9.0			<b></b>		

Appendix B. CHARACTERISTICS OF LMFBR SPENT FUEL, HIGH-LEVEL WASTE, AND STRUCTURAL MATERIAL WASTE Appendix B.l. Radioactivity of LMFBR Spent Fuel, High-Level Waste, and Structural Material Waste



Fig. B.1. Radioactivity of LMFBR spent core fuel as a function of decay time.

Time (years)	TOTAL	PU239	PU240	PU241	CS137	AM241	RH106	RU106	SR 90	PR144	CE144	NB 95
$\begin{array}{c} 1.000 E \div 01 \\ 1.000 E 0 \\ 0.000 E 01 \\ 1.000 E 02 \\ 3.000 E 02 \\ 1.000 E 03 \\ 1.000 E 03 \\ 1.000 E 04 \\ 3.000 E 04 \\ 1.000 E 05 \\ 3.000 E 05 \\ 1.000 E 05 \\ 1.000 E 05 \\ 1.000 E 05 \end{array}$	3.877E 07 9.583E 06 1.660E 06 1.700E 05 5.345E 04 2.680E 04 8.904E 03 3.286E 03 5.461E 02 2.004E 02 1.192E 02	6.101E 03 6.100E 03 6.100E 03 6.086E 03 6.086E 03 5.936E 03 4.622E 03 3.498E 03 3.498E 02 1.107E 00 8.230E-07	1.119E 04 1.120E 04 1.120E 04 1.089E 04 1.012E 04 3.896E 03 4.673E 02 2.793E-01 3.413E-06 3.394E-06	1.308E 06 1.252E 06 8.120F 05 1.067E 05 1.067E 00 3.265E 00 1.567E 00 3.067E-01 1.022E-03 8.423E-11 0.0	5.948E 05 5.826E 05 4.732E 05 5.915E 04 5.821E 02 5.506E-05 0.0 0.0 0.0 0.0 0.0	4.681E 03 6.520E 03 2.098E 04 4.202E 04 1.001E 04 1.572E 00 3.067E-01 1.022E-03 8.875E-11 0.0	3.144E 06 1.6694E 06 3.530E 03 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	3.144E 06 1.694E 06 3.530E 03 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	2+210E 05 2+163E 05 1+746E 05 2+050E 04 1+755E 02 1+019E-05 0+0 0+0 0+0 0+0	2.872E 06 1.2289E 06 4.256E 02 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	2.872E 06 1.2288E 06 4.254E 02 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	4.092E 06 1.892E 35 5.609E-11 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0
					Radio	pactivity (C	i/MTIHM)					
Time (years)	PU242	PU238	PM147	RU103	ZR 95	RH103M	NP237	PA233	U233	TH229	AC225	RA225
$\begin{array}{c} 1.000E-0.1\\ 1.000E 00\\ 1.000E 01\\ 1.000E 02\\ 3.000E 02\\ 1.000E 04\\ 3.000E 04\\ 1.000E 04\\ 1.000E 05\\ 3.000E 05\\ 1.000E 05\\ \end{array}$	$\begin{array}{c} \textbf{3.542E} & \textbf{01} \\ \textbf{3.542E} & \textbf{01} \\ \textbf{3.542E} & \textbf{01} \\ \textbf{3.542E} & \textbf{01} \\ \textbf{3.543E} & \textbf{01} \\ \textbf{3.543E} & \textbf{01} \\ \textbf{3.543E} & \textbf{01} \\ \textbf{3.483E} & \textbf{01} \\ \textbf{3.483E} & \textbf{01} \\ \textbf{2.964E} & \textbf{01} \\ \textbf{2.972E} & \textbf{01} \\ \textbf{5.911E} & \textbf{00} \end{array}$	2.728E 04 2.847E 04 2.697E 04 1.339E 04 2.865E 03 1.957E 01 1.365E-17 0.0 0.0 0.0	6.326E 05 5.008E 05 4.644E 04 2.187E-06 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	3.475E 06 1.056E 04 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	3.067E 06 8.713E 04 2.977E-11 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	3.134E 06 9.523E 03 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	$\begin{array}{c} 3.168 \pm -01\\ 3.186 \pm -01\\ 3.602 \pm -01\\ 1.504 \pm 00\\ 3.852 \pm 02\\ 8.042 \pm 02\\ 1.004 \pm 01\\ 9.982 \pm 00\\ 9.760 \pm 00\\ 9.760 \pm 00\\ 7.291 \pm 00\\ \end{array}$	$\begin{array}{c} 3.085 \pm -01\\ 3.186 \pm -01\\ 1.504 \pm 00\\ 3.852 \pm 00\\ 8.942 \pm 00\\ 1.004 \pm 01\\ 9.982 \pm 00\\ 9.760 \pm 00\\ 9.760 \pm 00\\ 7.291 \pm 00\\ \end{array}$	2.025E-05 2.165E-05 3.4866-05 3.869E-04 2.783E-03 2.212E-02 4.093E-01 1.213E 00 3.495E 00 6.942E 10 7.737E 00	$\begin{array}{c} 2 & 225 \pm -06\\ 2 & 227 \pm -06\\ 3 & 249 \pm -06\\ 3 & 690 \pm -06\\ 3 & 016 \pm -05\\ 7 & 654 \pm -04\\ 1 & 401 \pm -01\\ 8 & 164 \pm -01\\ 3 & 189 \pm 00\\ 7 & 012 \pm 00\\ 7 & 012 \pm 00\\ 7 & 753 \pm 00\\ \end{array}$	2.591E-06 2.227E-06 2.249E-06 3.690E-06 3.516E-05 7.654E-04 1.431E-01 8.164E-01 3.189E 00 7.912E 00 7.753E 00	$\begin{array}{c} 2.39  4E - 06 \\ 2.22  7E - 06 \\ 3.69  0E - 06 \\ 3.69  0E - 06 \\ 3.01  6E - 05 \\ 7.65  4E - 04 \\ 1.40  1E - 01 \\ 8.16  4E - 01 \\ 3.18  9E  00 \\ 7.31  2E  00 \\ 7.75  3E  00 \end{array}$

Table B.1. Radioactivity of LMFRR spent core fuel as a function of decay time

Radioactivity (Ci/MTIHM)

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Fig. B.2. Radioactivity of LMFBR spent core + axial blanket fuel as a function of decay time.

Table B.2. Radioactivity of LMFBR spent core + axial blanket fuel as a function of decay time

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Radioactivity (C	i/MTIHM)
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Time (years)	TOTAL	PU239	PU240	C\$137	PU241	AM241	RH106	RU106	SR 90	PR144	CE144	NB 95
1.000E-01 1.000E 00 1.000E 02 3.000E 02 1.000E 02 1.000E 04 3.000E 04 3.000E 04 3.000E 05 3.000E 05 1.000E 06	2.620E 07 6.409E 06 1.097E 06 1.131E 05 3.557E 04 1.817E 04 6.332E 03 2.428E 03 3.428E 03 1.324E 02 7.969E 01	4.642E 03 4.642E 03 4.630E 03 4.630E 03 4.630E 03 4.516E 03 3.511E 03 1.987E 03 2.6532E 02 8.390E-01 5.346E-07	$\begin{array}{cccccc} 7.375E & 03\\ 7.375E & 03\\ 7.375E & 03\\ 7.333E & 03\\ 7.333E & 03\\ 7.180E & 03\\ 5.667E & 03\\ 3.0680E & 02\\ 1.841E-01\\ 2.216E-06\\ 2.204E-06\end{array}$	3.981E 05 3.8899E 05 3.1666E 05 3.958E 04 3.897E 02 3.6866E-05 0.0 0.0 0.0 0.0 0.0	$\begin{array}{l} 8.509E \hspace{0.1cm} 05\\ 8.145E \hspace{0.1cm} 05\\ 5.281E \hspace{0.1cm} 03\\ 2.701E \hspace{0.1cm} 03\\ 2.120E \hspace{0.1cm} 00\\ 1.017E \hspace{0.1cm} 00\\ 1.992E-01\\ 6.638E-04\\ 5.470E-11\\ 0.0 \end{array}$	3.041E 03 4.237E 03 1.364E 04 2.733E 04 2.733E 04 2.000E 04 1.021E 00 1.992E-01 6.638E-04 5.763E-11 0.0	2.101E 06 1.131E 06 2.361E 03 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	2.101E 06 1.131E 06 2.361E 03 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	1.492E 05 1.461E 05 1.179E 05 1.384E 04 1.185E 02 6.884E-06 0.0 .0.0 0.0 0.0 0.0	1.936E 06 8.687E 05 2.868E 02 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	1.936E 06 8.687E 05 2.868E 02 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	2.778E 06 1.285E 05 4.491E-11 3.0 0.0 0.0 0.0 0.0 3.0 0.0 0.0 0.0 0.0

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Radioactivity (Ci/MTIHM)

Time (years) PU242	PM147	PU238	RU103	ZR 95	RH103M	NP237	PA233	U233	тн229	AC225	R4225
1.000E-01 2.300E 1.000E 00 2.300E 1.000E 01 2.300E 1.000E 02 2.301E 3.000E 02 2.301E 1.000E 03 2.298E 1.000E 04 2.262E 3.000E 04 2.182E 1.000E 05 1.924E 3.000E 05 1.346E 1.000E 06 3.839E	01 4.293E 05 01 3.398E 05 01 3.152E 04 01 0.848E-06 01 0.0 01 0.0 01 0.0 01 0.0 01 0.0 01 0.0 01 0.0 01 0.0	1.777E 04 1.854E 04 1.756E 04 8.721E 03 1.866E 03 1.872E 01 8.865E 18 0.0 0.0 0.0 0.0	2.351E 06 7.134E 03 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	2.084E 06 5.919E 04 2.023E-11 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	2.120E 06 6.432E 03 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	$\begin{array}{c} 2 \cdot 258E-01\\ 2 \cdot 269E-01\\ 2 \cdot 541E-01\\ 9 \cdot 979E-01\\ 2 \cdot 525E & 00\\ 5 \cdot 250E & 00\\ 5 \cdot 550E & 00\\ 6 \cdot 5511E & 00\\ 6 \cdot 366E & 00\\ 5 \cdot 966E & 00\\ 4 \cdot 756E & 00\\ \end{array}$	$\begin{array}{c} 2 \cdot 199 \pm -01\\ 2 \cdot 270 \pm -01\\ 2 \cdot 54 \pm -11\\ 9 \cdot 979 \pm -01\\ 2 \cdot 525 \pm 00\\ 5 \cdot 250 \pm 00\\ 6 \cdot 550 \pm 00\\ 6 \cdot 551 \pm 00\\ 6 \cdot 366 \pm 00\\ 5 \cdot 966 \pm 00\\ 4 \cdot 756 \pm 00\\ \end{array}$	1.348E-05 1.448E-05 2.385E-05 2.636E-04 1.835E-03 1.448E-02 2.670E-01 7.915E-01 2.280E00 4.527E00 5.046E00	1.446E-06 1.447E-06 2.442E-06 2.442E-06 5.017E-04 9.137E-02 5.326E-01 2.080E 00 4.574E 00 5.057E 00	$\begin{array}{c} 1.682E-06\\ 1.447E-06\\ 1.462E-06\\ 2.442E-06\\ 1.9998E-05\\ 5.017E-04\\ 9.137E-02\\ 5.326E-01\\ 2.080E 00\\ 4.574E 00\\ 5.057E 00\\ \end{array}$	1.555E-06 1.447E-06 1.462E-06 2.442E-06 1.999BE-05 5.017E-02 5.326E-01 2.080E 00 4.574E 00 5.057E 00

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Fig. B.3. Radioactivity of LMFBR spent radial blanket fuel as a function of decay time.

					Radi	oactivity (C	i/MTIHM)					
Time (years)	TOTAL	PU239	C\$137	SR 90	PU240	PR144	CE144	RH106	RU136	PN147	N8 95	ZR 95
$\begin{array}{c} 1.000 \pm -01\\ 1.000 \pm 00\\ 1.000 \pm 02\\ 3.000 \pm 02\\ 1.000 \pm 02\\ 1.000 \pm 03\\ 1.000 \pm 04\\ 3.000 \pm 04\\ 3.000 \pm 05\\ 3.000 \pm 05\\ 1.000 \pm 06\end{array}$	$\begin{array}{c} \textbf{3.661E}\\ \textbf{6.520E}\\ \textbf{055}\\ \textbf{6.548E}\\ \textbf{04}\\ \textbf{9.368E}\\ \textbf{03}\\ \textbf{2.802E}\\ \textbf{03}\\ \textbf{2.802E}\\ \textbf{03}\\ \textbf{2.802E}\\ \textbf{03}\\ \textbf{2.802E}\\ \textbf{03}\\ \textbf{2.82E}\\ \textbf{02}\\ \textbf{2.82E}\\ \textbf{02}\\ \textbf{7.342E}\\ \textbf{00}\\ \textbf{6.936E}\\ \textbf{00} \end{array}$	2.158E 2.158E 2.157E 2.157E 2.157E 2.097E 1.618E 9.090E 1.211E 3.830E- 6.705E-	3 4.246E 04   3 4.158E 04   3 3.378E 04   3 4.222E 03   3 4.159E 01   3 4.033E-06 06   3 0.00 0.0   02 0.0 0   01 0.0 0	$\begin{array}{cccc} 2.009E & 04\\ 1.966E & 04\\ 1.587E & 04\\ 1.863E & 03\\ 1.596E & 01\\ 9.266E & -07\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0$	3.792E 02 3.791E 02 3.788E 02 3.752E 02 3.673E 02 3.410E 02 1.313E 02 1.575E 01 9.420E-03 2.100E-11 1.510E-11	2.517E 05 1.129E 05 3.729E 01 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	2.517E 35 1.129E 35 3.729E 01 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	2.138E 05 1.151E 05 2.394E 02 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	2.138E 05 1.151E 05 2.394E 02 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	6.178E 04 4.893E 04 4.538E 03 2.137E-07 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	4.643E 05 2.134E 04 7.452E-12 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.	3.460E 05 9.826E 03 3.358E-12 0.0 3.9 3.0 3.0 3.0 3.3 0.0 0.0

Table B.3. Radioactivity of LMFBR spent radial blanket fuel as a function of decay time

Radioactivity (Ci/MTIHM)

Time (years)	RU103	RH 103M	CE141	Y 91	PU24 1	ZR 93	SM151	U238	P4234M	TH234	U234	RA226
1.000E-01 1.000E 02 1.000E 02 3.000E 03 1.000E 03 1.000E 04 3.000E 04 3.000E 04 3.000E 05 1.000E 05	3.526E 0 1.060E 0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	5 3.178E 05 3 9.617E 02 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	2.716E 05 2.456E 02 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	2.031E 05 4.133E 03 5.043E-14 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	5.103E 03 4.887E 03 3.169E 03 4.162E 01 2.745E-03 2.784E-06 1.336E-06 2.615E-07 8.668E-10 7.133E-17 0.0	3.325E-01 3.450E-01 4.433E-01 6.132E-01 6.120E-01 6.118E-01 6.093E-01 5.850E-01 5.850E-01 3.891E-01	6.702E02 8.639E02 8.060502 4.030E02 8.6639E01 3.938E-01 0.0 0.3 0.3 0.3 0.0	3.21 IE-01 3.21 IE-01	3. 21 2E-01 3. 21 1E-01 3. 21 0E-01	3.212E-01 3.211E-01 3.211E-01 3.211E-01 3.211E-01 3.211E-01 3.211E-01 3.211E-01 3.211E-01 3.211E-01 3.210E-01	1.112E-03 1.658E-03 6.919E-03 4.326E-02 7.892E-02 7.892E-02 9.802E-02 1.382E-01 1.382E-01 2.173E-01 3.068E-01	$\begin{array}{c} 5.571E-12\\ 1.233E-11\\ 6.462E-07\\ 1.352E-06\\ 1.352E-04\\ 5.352E-03\\ 2.105E-02\\ 1.721E-01\\ 3.002E-01\\ \end{array}$

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Fig. B.4. Radioactivity of LMFBR high-level waste as a function of decay time.

Table B.4.	Radioactivity of	LMFBR high-level	waste as a	function of	decay time
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					Radio	pactivity (C	i/MTIHM)					
Time — (years)	TOTAL	CS137	AN241	TC 99	SR 90	NP 239	AM243	PU239	RH106	RU106	ZR 93	PR144
$3 \cdot 500 = -01$ $1 \cdot 250 = 00$ $1 \cdot 025 = 01$ $1 \cdot 003 = 02$ $2 \cdot 0003 = 02$ $1 \cdot 000 = 04$ $3 \cdot 000 = 04$ $1 \cdot 000 = 05$ $3 \cdot 000 = 05$ $1 \cdot 000 = 06$	8.328E 0 3.078E 0 3.069E 0 4.110E 0 2.728E 0 1.778E 0 7.330E 0 2.982E 0 1.929E 0 1.929E 0	6 2.658E ( 6 2.604E ( 5 2.115E ( 3 2.601E ( 2 2.601E ( 2 2.460E ( 2 2.00 1 0.0 1 0.0 1 0.0 1 0.0	$\begin{array}{c} 5 & 2 \cdot 051E & 03\\ 5 & 2 \cdot 052E & 03\\ 5 & 2 \cdot 052E & 03\\ 6 & 1 \cdot 825E & 03\\ 2 & 1 \cdot 325E & 03\\ 5 & 4 \cdot 322E & 02\\ 6 \cdot 433E - 01\\ 1 \cdot 259E - 01\\ 4 \cdot 174E - 04\\ 3 \cdot 621E - 11\\ 0 \cdot 0\end{array}$	1.770E 01 1.770E 01 1.770E 01 1.770E 01 1.770E 01 1.765E 01 1.765E 01 1.714E 01 1.606E 01 1.279E 01 6.670E 00 6.836E-01	1.012E 05 9.901E 04 7.991E 04 9.383E 03 8.033E 01 4.666E-06 0.0 0.0 0.0 0.0	1.235E 02 1.235E 02 1.235E 02 1.223E 02 1.221E 02 1.201E 02 4.828E 01 7.379E 00 1.0379E 00 1.0379E 00 3.477E-07 3.370E-07	1.235E 02 1.2235E 02 1.2235E 02 1.223E 02 1.223E 02 1.224E 02 4.828E 01 7.379E 00 1.028E-02 3.477E-07 3.370E-07	1.865E 01 1.865E 01 1.870E 01 1.906E 01 2.158E 01 3.367E 01 2.767E 01 4.114E 00 1.302E-02 3.370E-07	1.188E 06 6.398E 05 1.313E 03 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	1.188E 06 6.398E 05 1.307E 03 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	1.845E 00 1.912E 00 2.440E 00 3.339E 00 3.348E 00 3.334E 00 3.334E 00 3.334E 00 3.303E 00 2.923E 00 2.129E 00	1.057E 06 4.744E 35 1.5566E 02 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0

Radioactivity (Ci/MTIHM)

Time (years) <b>CE14</b>	4 CS135	PH147	PU240 NB 95	SM151 ZR 95	NP237 P4233	C\$134 U233 TH229
$\begin{array}{cccccccccccccccccccccccccccccccccccc$	7E 06 2.103 4E 05 2.103 6E 02 2.103 2.103 2.103 2.102 2.097 2.084 2.041 1.921 1.556	E 00 2.767E 05 E 00 2.182E 05 E 00 2.182E 05 E 00 2.023E 05 E 00 0.02 E 00 0.0 E 00 0.0 E 00 0.0 E 00 0.0 E 00 0.0 E 00 0.0	2.410E 01 9.930E 05 52.487E 01 3.402E 04 3.124E 01 1.175E-11 4.590E 01 0.0 4.513E 01 0.0 1.624E 01 0.0 1.948E 00 0.0 1.165E-03 0.0 7.532E-09 0.0	4.784E 03 5.452E 05   4.751E 03 1.549E 04   4.433E 03 5.291E-12 2.216E 03 0.0   4.750E 02 0.0 2.164E 03 0.0   0.0 0.0 0.0 0.0 0.0   0.0 0.0 0.0 0.0 0.0   0.0 0.0 0.0 0.0 0.0   0.0 0.0 0.0 0.0 0.0   0.0 0.0 0.0 0.0 0.0   0.0 0.0 0.0 0.0 0.0	1.699E-01 1.695E-01 1.705E-01 1.705E-01 2.334E-01 2.334E-01 3.345E-01 3.345E-01 5.151E-01 5.151E-01 6.033E-01 6.033E-01 6.016E-01 6.016E-01 5.886E-01 5.886E-01 5.517E-01 5.517E-01 4.398E-01 4.398E-01	$\begin{array}{cccccccccccccccccccccccccccccccccccc$

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Fig. B.5. Radioactivity of LMFBR structural material waste as a function of decay time.

		·.	······							Ra	adio	pactivity (C	i/MTIHM)					
Time (years) 3.500E-01	TOTAL 1.588E	05	NI 59	00	MN 54 5.245E 0	4	NI 63 2.335E	02	FE 1.	55 158E	04	CD 58 8-641E 04	PU239	PU240 2.403E 00	CD 60 2.153E 03	AM241 1.069E 00	TC 99 8.329E-02	C 14 1.132E 00
1.250E 00 1.025E 01 1.003E 02 1.000E 03 1.000E 03 1.000E 04 1.000E 04 3.000E 04	4.183 2.003 2.1.522 4.099 1.106 5.935 5.0579 5.1.579 5.1.579 5.3.152 5.5.255 5.3.152 5.3.152 5.3.152 5.5.255 5.3.152 5.5.255 5.5.255 5.5.255 5.5.255 5.5.255 5.5.255 5.5.5555 5.5.5555 5.5.5555 5.5.5555 5.5.5555 5.5.55555 5.5.55555 5.5.555555	03 02 01 00 00 00	3.238E 3.238E 3.236E 3.230E 3.230E 2.970E 2.497E 1.362E 2.407E	00 00 00 00 00 00 00	2.530E U 1.724E 0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	ī	2.319E 2.167E 1.100E 2.438E 1.250E 0.0 0.0 0.0	02 02 01 01	83000000	111E 270E 142E- 0 0 0 0 0 0	03	3.655E 03 3.6604E-11 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	1.865E 00 1.865E 00 1.850E 00 1.850E 00 1.814E 00 1.408E 00 7.961E-01 1.062E-01 3.378E-04	2.403E 00 2.388E 00 2.338E 00 2.338E 00 2.171E 00 8.361E-01 1.003E-01 5.996E-05 7.008E-10	1.912E 03 5.854E 02 4.229E-03 1.590E-14 0.0 0.0 0.0 0.0 0.0	1.444E 00 4.392E 00 8.667E 00 6.344E 00 2.065E 00 3.229E-04 6.297E-05 2.088E-07 1.812E-14	6.329E-02 8.328E-02 8.326E-02 8.321E-02 8.302E-02 8.302E-02 7.554E-02 6.015E-02 3.138E-02	1.131E 00 1.130E 00 1.118E 00 1.091E 00 1.003E 00 3.375E-01 3.002E-02 6.300E-06 1.953E-16
1.000E 00	5 .3.026E-	02	5.607E	-04	00	·.	<b>0</b> • 0		0.	0		0.0	1.692E-10	6.969E-10	0.5	0.0	3.216E-03	0.0

Table B.5. Radioactivity of LMFBR structural material waste as a function of decay time

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Radioactivity (Ci/MTIHM)

3.500E-01 1.330E 02 2.667E 02 2.749E-01 4.585E-10 4.599E-10 4.599E-10 4.599E-10 4.599E-10 4.599E-10 4.599E-10 4.589FE-09 4.500 1.250E 00 1.302E 02 2.554E 02 2.749E-01 4.588E-10 8.658E-10 8.658E-10 8.658E-10 8.658E-10 8.658E-10 8.058E-10 8.058E-1	ears)	CS137	PU241	NB 94	TH229	AC225	RA225	FR221	AT217	81213	PB209	U233	P0213
1.000E 04 0.0 3.218E-04 1.954E-01 2.921E-05 2.	5000000 2500000 025000 0030020 0000000 0000000 0000000 000000 000000	1.330E 02 1.302E 02 1.058E 02 1.322E 01 1.301E-01 1.231E-08 0.0 0.0 0.0 0.0	2.667E 02 2.554E 02 2.1656E 02 2.176E 00 8.531E-04 6.704E-04 3.218E-04 6.297E-05 2.088E-07 1.720E-14 0.0	$\begin{array}{c} 2.749E-01\\ 2.749E-01\\ 2.748E-01\\ 2.740E-01\\ 2.721E-01\\ 2.657E-01\\ 1.954E-01\\ 9.869E-02\\ 9.041E-03\\ 9.828E-06\\ 4.091E-16\end{array}$	$\begin{array}{c} 4.585 \Xi - 10 \\ 4.588 \Xi - 10 \\ 4.641 \Xi - 10 \\ 8.058 \Xi - 10 \\ 6.603 \Xi - 09 \\ 1.619 \Xi - 07 \\ 2.921 \Xi - 05 \\ 1.701 \Xi - 04 \\ 6.642 \Xi - 04 \\ 1.460 \Xi - 03 \\ 1.615 \Xi - 03 \end{array}$	$\begin{array}{c} 4 \cdot 599 E - 10 \\ 4 \cdot 588 E - 10 \\ 8 \cdot 058 E - 10 \\ 6 \cdot 603 E - 09 \\ 1 \cdot 619 E - 07 \\ 2 \cdot 921 E - 05 \\ 1 \cdot 701 E - 04 \\ 6 \cdot 642 E - 04 \\ 1 \cdot 44 \cdot 0E - 03 \\ 1 \cdot 615 E - 03 \end{array}$	$\begin{array}{c} 4.589E-10\\ 4.588E-10\\ 8.058E-10\\ 6.058E-10\\ 6.058E-10\\ 6.03E-09\\ 1.619E-07\\ 2.921E-05\\ 1.701E-04\\ 6.642E-04\\ 1.460E-03\\ 1.615E-03\\ \end{array}$	$\begin{array}{c} \textbf{4} .599E-10\\ \textbf{4} .588E-10\\ \textbf{8} .641E-10\\ \textbf{8} .058E-10\\ \textbf{6} .603E-09\\ \textbf{1} .619E-07\\ \textbf{2} .921E-05\\ \textbf{1} .701E-04\\ \textbf{6} .642E-03\\ \textbf{1} .460E-03\\ \textbf{1} .615E-03\\ \end{array}$	$\begin{array}{c} 4.599 \\ \pm .588 \\ \pm .508 \\ \pm .588 \\ \pm .588$	$\begin{array}{c} 4.599E-10\\ 4.588E-10\\ 6.641E-10\\ 8.058E-10\\ 6.632E-09\\ 1.619E-07\\ 2.921E-05\\ 1.701E-04\\ 6.642E-04\\ 1.460E-03\\ 1.615E-03\\ \end{array}$	$\begin{array}{c} 4.599E-10\\ 4.588E-10\\ 4.641E-10\\ 8.058E-10\\ 6.603E-09\\ 1.619E-07\\ 2.921E-05\\ 1.701E-04\\ 6.642E-04\\ 1.460E-03\\ 1.615E-03\\ \end{array}$	4.587E-09 4.952E-09 8.465E-09 8.905E-08 6.008E-07 4.653E-06 8.528E-05 2.528E-05 2.528E-04 7.279E-04 1.446E-03 1.611E-03	$\begin{array}{c} 4.500 E-10\\ 4.489 E-10\\ 4.541 E-10\\ 7.884 E-10\\ 6.461 E-09\\ 1.584 E-07\\ 2.858 E-05\\ 1.5654 E-05\\ 1.6564 E-03\\ 1.580 E-03\\ 1.580 E-03\\ \end{array}$

Appendix B.2. Thermal Power of LMFBR Spent Fuel, High-Level Waste, and Structural Material Waste

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Fig. B.6. Thermal power of LMFBR spent core fuel as a function of decay time.

	Thermal power (W/MTIHM)												
Time (years)	TOTAL	PU239	AM241	PU240	RH106	PU238	PR144	CS137	SR 90	CN242	P0213	NB 95	
1.000E-01 1.000E 00 1.000E 02 3.000E 02 1.000E 03 1.000E 04 3.000E 04 1.000E 05 3.000E 05 3.000E 05	1.638E 05 3.990E 04 4.736E 03 2.627E 03 1.660E 03 8.427E 02 2.700E 02 9.849E 01 1.441E 01 4.180E 00 2.642E 00	1.880E 02 1.880E 02 1.880E 02 1.880E 02 1.805E 02 1.830E 02 1.424E 02 8.073E 01 1.078E 01 3.413E-02 2.536E-08	1.555E 02 2.166E 02 6.970E 02 1.395E 03 1.022E 03 3.326E 02 5.222E-02 1.019E-02 3.396E-05 2.948E-12 0.0	3.485E 02 3.485E 02 3.487E 02 3.465E 02 3.465E 02 3.149E 02 1.213E 02 1.415E 02 1.456E 01 8.700E-03 1.063E-07 1.057E-07	3.016E 04 1.624E 04 3.386E 01 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	9.040E 02 9.435E 02 8.937E 02 4.436E 02 9.495E 01 6.485E-01 4.525E-19 0.0 0.0 0.0 0.0	2.111E 04 9.471E 03 3.127E 00 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	1.474E 03 1.443E 03 1.465E 02 1.465E 02 1.442E 03 1.364E-07 0.0 0.0 0.0 0.0 0.0	7.439E 02 7.251E 02 5.853E 02 6.871E 01 5.882E-01 3.416E-38 0.0 0.7 0.0 0.0 0.0	1.335E 04 3.316E 03 1.282E 01 8.502E 00 3.415E 00 1.403E-01 2.114E-19 0.0 0.0 0.0 0.0	$\begin{array}{c} 1 & 282 E - 07 \\ 1 & 103 E - 07 \\ 1 & 113 E - 07 \\ 1 & 827 E - 07 \\ 1 & 994 E - 06 \\ 3 & 790 E - 06 \\ 6 & 938 E - 03 \\ 4 & 042 E - 02 \\ 1 & 379 E - 01 \\ 3 & 471 E - 01 \\ 3 & 839 E - 01 \\ \end{array}$	1.962E 04 9.071E 03 3.170E-13 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.	

Table B.6. Thermal power of LMFBR spent core fuel as a function of decay time

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Thermal power (W/MTIHM)

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1.000E-01 1.105E-07 1.046E 00 7.058E 02 9.997E-08 9.048E-08 9.684E-03 1.554E 04 3.194E 03 6.838E-08 5.885E-07 1.357E 04 1.163E 04 1.000E 00 9.503E-08 1.046E 00 6.820E 02 8.595E-08 7.779E-08 9.738E-03 4.413E 02 2.360E 03 6.812E-08 6.292E-07 2.486E-04 3.534E 01 1.000E 01 9.593E-08 1.046E 00 1.632E 02 8.680E-08 7.657E-08 1.101E-02 1.508E-13 1.146E 02 6.880E-08 1.013E-06 0.3 1.000E 02 1.575E-07 1.046E 00 1.542E 01 1.424E-07 1.289E-07 4.596E-02 0.0 8.312E-12 1.129E-07 1.125E-05 0.0 3.0 1.000E 02 1.207E-06 1.046E 00 7.306E-03 1.164E-06 1.053E-06 1.177E-01 0.0 0.0 9.226E-07 8.088E-05 0.0 3.0 1.000E 03 3.266E-05 1.045E 00 1.200E-11 2.954E-05 2.673E-05 2.456E-01 0.0 0.0 9.226E-07 8.088E-05 0.0 0.0 1.000E 04 3.485E-02 9.924E-01 1.98E-11 3.151E-02 2.852E-02 3.051E-01 0.0 0.0 2.426E-03 1.190E-02 0.0 0.0 1.000E 04 3.485E-02 9.924E-01 1.198E-11 3.151E-02 2.852E-02 3.051E-01 0.0 0.0 9.757E-02 1.016E-01 0.0 0.0 1.000E 05 1.361E-01 8.753E-01 1.198E-11 1.231E-01 1.114E-01 2.993E-01 0.0 0.0 2.145E-01 2.317E-01 0.0 0.0 3.000E 05 2.992E-01 6.117E-01 1.198E-11 2.706E-01 2.796E-01 0.0 0.0 2.145E-01 2.249E-02 3.03 0.0 0.0 3.000E 05 2.992E-01 6.117E-01 1.198E-11 2.992E-01 2.796E-01 0.0 0.0 0.0 2.145E-01 2.249E-01 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	Time (years)	AT217	PU242	CM244	FR221	AC225	NP237	ZR 95	CS134	TH229	U233	LA140	RU103
	1.000E-01 1.000E 00 1.000E 01 1.000E 02 3.000E 02 1.000E 03 1.000E 04 1.000E 04 1.000E 05 3.000E 05	1.105E-07 9.503E-08 9.593E-08 1.575E-07 1.287E-06 3.266E-05 5.980E-03 3.485E-02 1.361E-01 2.992E-01 3.309E-01	1.046E 00 1.046E 00 1.046E 00 1.046E 00 1.046E 00 1.045E 00 1.026E 00 9.924E-01 8.753E-01 6.117E-01 1.746E-01	7.058E 02 6.820E 02 4.832E 02 1.542E 01 7.306E-03 1.200E-11 1.198E-11 1.198E-11 1.198E-11 1.198E-11 1.198E-11 1.189E-11	$\begin{array}{c} 9.997E-08\\ 8.595E-08\\ 8.680E-08\\ 1.424E-08\\ 1.424E-07\\ 1.164E-06\\ 2.954E-03\\ 3.151E-02\\ 1.231E-01\\ 2.706E-01\\ 2.992E-01\\ \end{array}$	9.048E-08 7.779E-08 7.857E-08 1.2857E-06 2.673E-05 2.673E-05 2.852E-02 1.114E-01 2.449E-01 2.4708E-01	9.684E-03 9.738E-03 1.101E-02 4.596E-02 1.177E-01 2.458E-01 3.070E-01 3.051E-01 2.983E-01 2.796E-01 2.229E-01	1.554E 04 4.413E 02 1.508E-13 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.	3.194E 93 2.360E 03 1.146E 02 8.312E-12 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	6.838E-08 6.812E-08 6.880E-08 1.129E-07 9.226E-07 2.342E-05 4.287E-03 2.498E-02 9.757E-02 2.145E-01 2.372E-01	5.885E-07 6.292E-07 1.013E-05 8.088E-05 6.432E-04 1.190E-02 3.528E-02 1.016E-01 2.317E-01 2.249E-01	1.357E 04 2.486E-04 0.3 0.0 0.3 0.0 0.3 0.0 0.0 0.0 0.0 0.0	1.163E 04 3.534E 01 3.0 3.0 3.3 C.0 0.0 0.0 0.0 3.3 3.3 3.0 3.0



Fig. B.7. Thermal power of LMFBR spent core + axial blanket fuel as a function of decay time.

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Table B.7. Thermal power of LMFBR spent core + axial blanket fuel as a function of decay time

Thermal	power	(W/MTIHM)

Time (years) 1.000E-01 1.000E 02 3.000E 02 1.000E 03 1.000E 03 1.000E 04 1.000E 04 1.000E 05 3.000E 05 1.000E 06	TOTAL 1.103E 05 2.666E 04 3.146E 03 1.737E 03 1.103E 03 5.709E 02 1.922E 02 7.295E 01 1.056E 01 2.750E 00 1.759E 00	PU239 1.431E 02 1.431E 02 1.431E 02 1.427E 02 1.391E 02 1.391E 02 1.082E 02 6.125E 01 8.177E 00 2.586E-02 1.647E-08	AM241 1.011E 02 4.532E 02 9.078E 02 6.644E 02 2.162E 02 3.391E-02 6.615E-03 2.205E-05 1.914E-12 0.0	PU240 2.297E 02 2.298E 02 2.283E 02 2.283E 02 2.283E 02 2.235E 02 2.076E 02 7.993E 01 9.588E 01 9.588E 01 9.588E 00 5.733E-03 6.900E-08 6.862E-08	RH106 2.015E 04 1.085E 04 2.264E 01 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	PU238 5.889E 02 6.145E 02 5.820E 02 2.889E 02 6.183E 01 4.219E-01 2.939E-19 0.0 0.0 0.0	PR144 1.423E 04 6.384E 03 2.108E 00 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	CS137 9.861E 02 7.845E 02 7.845E 02 9.805E 01 9.635E 01 9.635E 01 9.635E 00 0.0 0.0 0.0 0.0 0.0 0.0	SR 90 5.002E 02 4.895E 02 3.951E 02 4.639E 01 3.973E 01 2.3J8E 08 0.0 0.0 0.0 0.0 0.0 0.0	CM242 8,670E 03 2,153E 03 8,324E 00 5,521E 00 9,112E-02 1,373E-19 0.0 0.0 0.0 0.0	PD213 6.330E-08 7.162E-08 7.237E-08 1.209E-07 9.894E-07 2.484E-05 4.526E-03 2.637E-02 1.030E-01 2.264E-01 2.504E-01	NB 95 1.333E 04 6.164E 02 2.154E-13 0.0 0.0 0.0 0.0 3.0 0.0 3.0 0.0 3.0 0.0 3.0 0.0 0
Time		·			Thermal	power (W/M1	[IHM]					<u></u>
(years)	AT217	CM244	PU242	FR221	AC225	ZR 95	NP237	C\$134	TH229	U233	LA140	RU103
1.000E-01 1.000E 00 1.000E 02 3.000E 02 1.000E 03 1.000E 03 1.000E 04 1.000E 05 3.000E 05 3.000E 05	7.180E-08 6.174E-08 6.237E-07 8.525E-07 2.141E-05 3.901E-03 2.273E-02 8.874E-02 1.952E-01 2.158E-01	4.583E )2 4.428E 02 3.137E 02 1.001E 01 4.743E-03 7.794E-12 7.783E-12 7.761E-12 7.766E-12 7.766E-12 7.72E-12	6.793E-01 6.793E-01 6.793E-01 6.794E-01 6.794E-01 6.678E-01 6.678E-01 6.444E-01 5.684E-01 3.973E-01 1.134E-01	6.494E-08 5.583E-08 5.641E-08 9.426E-08 7.712E-07 1.936E-03 3.528E-03 2.055E-02 6.028E-02 1.765E-01 1.952E-01	5.877E-08 5.053E-08 5.106E-08 8.525E-08 6.980E-07 1.753E-05 3.193E-03 1.861E-02 7.267E-02 1.598E-01 1.766E-01	1.056E 04 2.999E 02 1.024E-13 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.	6.901E-03 6.937E-03 7.765E-02 7.717E-02 1.605E-01 2.002E-01 1.990E-01 1.990E-01 1.946E-01 1.453E-01 1.453E-01	$\begin{array}{c} 2.092E & 03\\ 1.545E & 03\\ 7.501E & 01\\ 5.442E-12\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.$	4.423E-08 4.426E-08 4.472E-08 7.470E-08 6.113E-07 1.535E-05 2.796E-03 1.629E-02 6.364E-02 1.399E-01 1.5347E-01	3.918E-07 4.208E-07 6.933E-07 7.576E-06 5.335E-05 4.208E-04 7.761E-03 2.301E-02 6.627E-02 1.316E-01 1.467E-01	9.231E 03 1.691E-04 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	7.867E 03 2.387E 01 3.3 0.0 0.0 0.0 0.0 0.0 0.0 0.0 3.3 3.3





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					Thermal	power (W/MT	IHM)					
Time — (years)	TOTAL	PU239	RH106	C\$137	PR144	PU240	SR 90	NB 95	ZR 95	LA140	P0214	FU103
1.000E-01 1.000E 00 1.000E 02 3.000E 02 1.000E 02 1.000E 03 1.000E 04 3.000E 05 3.000E 05 1.000E 05	1.430E 04 2.639E 03 2.427E 02 1.031E 02 8.199E 01 7.649E 01 7.649E 01 3.598E 01 3.798E 00 1.123E-01 1.310E-01	6.647E 01 6.647E 01 6.647E 01 6.647E 01 6.595E 01 6.462E 01 6.462E 01 2.802E 01 3.731E 00 1.181E-02 2.067E-11	2.050E 03 1.104E 03 2.296E 00 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	1.052E 02 1.030E 02 8.364E 01 1.046E 01 1.030E-01 9.745E-09 0.0 0.0 0.0 0.0 0.0 0.0 0.0	1.850E 03 8.297E 02 2.741E-01 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	1.181E 01 1.181E 01 1.179E 01 1.168E 01 1.068E 01 1.062E 01 4.0639E 00 4.9035E-01 2.933E-04 6.537E-13 4.702E-13	e.734E 01 6.590E 01 5.320E 01 5.3245E 00 5.344E-02 3.106E-09 0.0 0.0 0.0 0.0 0.0 0.0	2.227E 03 1.024E 02 3.576E-14 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	1.753E 03 4.977E 01 1.731E-14 3.0 0.0 0.0 0.0 0.0 3.3 3.3 0.0 0.0 3.3	1.549E 03 2.835E-05 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	3.277E-10 5.782E-13 2.999E-11 1.607E-08 2.992E07 4.883E-06 2.4884E-04 3.150E-03 7.990E-03 1.394E-02	1.179E 03 3.545E 00 0.0 0.0 0.0 0.0 0.0 3.0 3.0 0.0
Time -		······							<u> </u>	<u></u>		
(years)	P0218	RN222	AN241	P0210	U234	RA226	TH230	U238	PU238	Y 91	CS134	CE144
1.000E-01 1.000E 00 1.000E 01 1.000E 02 1.000E 02 1.000E 03 1.000E 04 1.000E 04 3.000E 05 1.000E 05	1.991E-13 4.470E-13 2.341E-11 1.255E-08 2.336E-07 3.812E-08 1.939E-04 7.631E-04 2.460E-03 6.236E-03 1.088E-02	1.820E-13 4.087E-13 2.141E-11 1.147E-08 2.136E-07 3.486E-06 1.773E-04 6.977E-04 4.249E-03 5.703E-03 9.947E-03	$\begin{array}{c} 2.148E-01\\ 4.537E-01\\ 2.334E-00\\ 5.113E 00\\ 3.744E 00\\ 1.219E 00\\ 7.017E-07\\ 8.685E-09\\ 2.880E-11\\ 2.497E-18\\ 0.0\end{array}$	9.386E-12 1.526E-11 1.443E-11 5.349E-09 1.588E-07 3.371E-06 1.715E-04 6.745E-04 6.745E-03 5.515E-03 9.623E-03	3.201E-05 4.774E-05 1.992E-04 1.246E-03 2.051E-03 2.273E-03 2.449E-03 2.449E-03 2.824E-03 3.980E-03 6.259E-03 8.836E-03	1.608E-13 3.561E-13 1.866E-11 9.994E-09 1.861E-07 3.037E-06 1.545E-04 6.079E-04 1.960E-03 8.668E-03	3.428E-10 6.601E-10 1.056E-08 6.346E-07 3.726E-05 1.746E-05 1.967E-04 5.923E-04 1.914E-03 4.864E-03 8.494E-03	6.141E-03 8.141E-03 8.141E-03 8.141E-03 8.141E-03 8.141E-03 8.141E-03 8.141E-03 8.141E-03 8.141E-03 8.141E-03	$\begin{array}{c} 7.399E & 00\\ 7.064E & 00\\ 6.583E & 00\\ 3.234E & 00\\ 6.664E-01\\ 2.686E-01\\ 3.682E-23\\ 7.492E-23\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ \end{array}$	7.295E 02 1.485E 01 1.811E-16 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.	9.490E 01 7.012E C1 3.403E 00 0.2.469E-13 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.	1.669E 02 7.492E 01 2.473E-02 3.0 3.0 3.0 3.0 3.0 3.0 5.0 5.0 5.0 5.0

Table B.8. Thermal power of LMFBR spent radial blanket fuel as a function of decay time

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Fig. B.9. Thermal power of LMFBR high-level waste as a function of decay time.

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	Thermal power (W/MTIHM)											
Time — (years)	TOTAL	AM241	C 5137	PU239	RH136	AM243	PR144	SR 90	CM244	PU240	P0213	CN242
3.500E-01 1.255E 00 1.025E 01 1.003E 02 3.003E 02 1.000E 03 1.000E 04 3.000E 05 3.000E 05 1.000E 06	3.779E 04 1.379E 04 1.203E 03 1.866E 02 5.671E 01 3.337E 00 1.274E 00 2.641E-01 1.708E-01 1.326E-01	6.813E 01 6.816E 01 6.063E 01 4.402E 01 1.436E 01 2.137E-02 4.182E-03 1.386E-05 1.203E-12 0.0	6.585E 02 6.449E 02 5.238E 02 6.547E 01 6.443E-01 6.092E-08 0.0 0.0 0.0 0.0 0.0	5.747E-01 5.748E-01 5.768E-01 5.873E-01 6.651E-01 1.638E-01 1.038E 00 8.527E-01 1.268E-01 4.012E-04 1.039E-08	1.139E 04 6.136E 03 1.259E 01 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	3.970E 00 3.970E 00 3.966E 00 3.966E 00 3.614E 00 1.552E 00 2.372E-01 3.305E-04 1.118E-08 1.083E-08	7.772E 03 3.487E 03 1.151E 00 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	3.391E 02 3.318E 02 2.679E 02 3.144E 01 2.692E-01 1.564E-08 0.0 0.0 0.0 0.0 0.0	2.870E 02 2.773E 02 1.965E 02 6.271E 00 2.971E-03 3.148E-14 2.468E-14 2.468E-14 2.468E-14 2.640E-14 2.791E-14	7.504E-01 7.743E-01 9.728E-01 1.429E 00 1.415E 00 1.313E 00 5.053E-01 6.067F-02 3.627E-05 2.345E-10 2.480E-10	1.806E-08 2.291E-10 4.999E-10 1.983E-08 2.112E-07 3.289E-06 4.263E-03 2.445E-03 2.445E-03 2.994E-02 2.315E-02	3.74 3E 03 9.307E 02 5.257E 00 3.487E 00 1.401E 00 5.756E-02 8.672E-20 0.0 0.0 3.0
					Thermal	power (W/M)	TIHM)					
Time — (years)	AT217	NP 237	FR221	AC225	N8 95	CS134	TH229	U233	PU238	ZR 95	SB126M	NP239
$\begin{array}{c} 3.500 \pm -01\\ 1.250 \pm 0.0\\ 1.025 \pm 0.0\\ 1.003 \pm 0.2\\ 3.003 \pm 0.2\\ 1.000 \pm 0.3\\ 1.000 \pm 0.4\\ 3.000 \pm 0.4\\ 3.000 \pm 0.5\\ 3.000 \pm 0.5\\ 1.000 \pm 0.5\\ 1.000 \pm 0.6\\ \end{array}$	1.557E-08 1.975E-10 3.533E-10 1.709E-08 1.820E-07 2.835E-04 2.107E-03 8.210E-03 1.805E-02 1.995E-02	5.193E-03 5.211E-03 5.394E-03 1.022E-02 1.574E-02 1.844E-02 1.849E-02 1.6839E-02 1.686E-02 1.344E-02	1.408E-08 1.786E-10 3.196E-10 1.546E-08 1.646E-07 2.5564E-06 3.323E-04 1.906E-03 7.426E-03 1.632E-02 1.805E-02	1.274E-08 1.617E-10 2.892E-10 1.399E-08 1.490E-07 2.320E-06 3.008E-04 1.725E-03 6.721E-03 1.477E-02 1.633E-02	4.763E 03 1.632E 02 5.635E-14 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.	1.250E 03 9.234E 02 4.482E 01 3.251E-12 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	$\begin{array}{c} 1.402E-10\\ 1.416E-10\\ 2.533E-10\\ 1.225E-08\\ 1.305E-07\\ 2.032E-06\\ 2.634E-04\\ 1.511E-03\\ 5.886E-03\\ 1.294E-02\\ 1.430E-02\\ 1.430E-02\\ \end{array}$	3.473E-09 2.524E-08 2.237E-07 2.574E-06 9.856E-06 4.9904E-05 7.237E-04 2.130E-03 1.217E-02 1.356E-02	4.772E 00 1.760E 01 2.069E 01 1.210E 01 3.954E 00 1.270E-01 1.856E-19 0.0 0.0 0.0	2.761E 03 7.844E 01 2.6680E-14 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.	2.618E-02 2.618E-02 2.617E-02 2.616E-02 2.612E-02 2.612E-02 2.642E-02 2.442E-02 2.442E-02 2.442E-02 2.442E-02 3.273E-03 2.558E-05	$\begin{array}{c} 2.985 \pm -01\\ 2.985 \pm -01\\ 2.985 \pm -01\\ 2.957 \pm -01\\ 2.957 \pm -01\\ 1.167 \pm -01\\ 1.167 \pm -01\\ 1.784 \pm -02\\ 2.489 \pm -05\\ 8.406 \pm -12\\ 8.147 \pm -10\\ \end{array}$

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Fig. B.10. Thermal power of LMFBR structural material waste as a function of decay time.

Thermal power (W/MTIHM)												
Time (years)	TOTAL	PU239	NI 59	CO 60	MN 54	AM241	PU240	CO 58	FE 55	PU238	P0213	AT217
3.500E-01 1.250E 0C 1.025E 01 1.003E 02 3.003E 02 1.000E 03 1.000E 04 3.000E 04 3.000E 05 1.000E 05 1.000E 06	8.490E 02 1.960E 02 1.13960E 02 1.1399 01 6.352E-01 3.972E-01 9.176E-02 4.527E-02 1.282E-02 2.438E-03 5.857E-04	5.749E-02 5.749E-02 5.747E-02 5.747E-02 5.733E-02 5.591E-02 4.341E-02 2.453E-02 3.273E-03 1.041E-05 5.214E-12	2.062E-02 2.062E-02 2.061E-02 2.056E-02 2.056E-02 2.056E-02 2.056E-02 1.891E-02 1.891E-02 1.590E-03 1.532E-03 3.570E-06	3.319E 01 2.949E 01 9.026E 00 6.521E-05 2.452E-16 0.0 0.0 0.0 0.0 0.0 0.0	2.612E 02 1.263E 02 8.583E-02 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	3.550E-02 4.796E-02 1.459E-01 2.879E-01 2.879E-01 2.107E-01 6.860E-02 1.073E-05 2.092E-06 6.936E-09 6.020E-16 0.0	7.481E-02 7.484E-02 7.484E-02 7.484E-02 7.436E-02 7.436E-02 2.6062-02 2.603E-02 3.123E-03 1.867E-06 2.182E-11 2.170E-11	5.174E 02 2.068E 01 2.157E-13 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.	1.593E 01 1.253E 01 1.137E 0 4.321E-11 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	1.912E-01 1.962E-01 1.8649E-02 1.964E-02 1.9364E-02 1.9364E-02 4.332E-23 0.0 0.0 0.0 0.0	2.277E-11 2.272E-11 2.298E-11 3.990E-11 3.229E-10 8.018E-09 1.446E-06 8.421E-06 8.289E-05 7.230E-05 7.993E-05	1.963E-11 1.958E-11 1.981E-11 3.439E-11 2.818E-10 6.911E-09 1.246E-06 7.258E-05 6.232E-05 6.891E-05
		-			Thermal	power (W/M <sup>-</sup>	TIHM)					
Time (years)	FR221	AC 225	TH229	NP237	U233	PU242	NI 63	CS137	NB 94	SR 90	U236	P0214
$\begin{array}{c} 3.500 \pm -01\\ 1.250 \pm 00\\ 1.025 \pm 01\\ 1.003 \pm 02\\ 3.003 \pm 02\\ 1.000 \pm 03\\ 1.000 \pm 04\\ 3.000 \pm 04\\ 1.000 \pm 05\\ 1.000 \pm 05\\ 1.000 \pm 06\end{array}$	1.775E-11 1.771E-11 1.791E-11 2.549E-10 6.250E-09 1.127E-06 6.564E-05 5.636E-05 6.232E-05	$\begin{array}{c} 1.607E-11\\ 1.603E-11\\ 2.615E-11\\ 2.815E-11\\ 2.307E-10\\ 5.657E-09\\ 1.020E-06\\ 5.941E-06\\ 5.101E-05\\ 5.101E-05\\ 5.641E-05\\ \end{array}$	1.403E-11 1.404E-11 1.420E-11 2.465E-11 2.020E-10 4.954E-09 8.935E-07 5.203E-06 2.032E-05 4.468E-05 4.940E-05	$\begin{array}{c} 2.598 \pm -06\\ 2.609 \pm -06\\ 2.879 \pm -06\\ 1.010 \pm -05\\ 5.132 \pm -05\\ 6.392 \pm -05\\ 6.354 \pm -05\\ 5.823 \pm -05\\ 5.823 \pm -05\\ 4.641 \pm -05\\ \end{array}$	$\begin{array}{c} 1.333 = -10\\ 1.442 = -10\\ 2.461 = -10\\ 2.588 = -09\\ 1.747 = -08\\ 1.353 = -07\\ 2.479 = -06\\ 7.348 = -06\\ 2.116 = -05\\ 4.202 = -05\\ 4.684 = -05\\ \end{array}$	2.149E-04 2.149E-04 2.149E-04 2.149E-04 2.149E-04 2.147E-04 2.1147E-04 2.1147E-04 2.038E-04 1.256E-04 3.585E-05	9.273E-02 9.210E-02 8.6606E-02 9.681E-03 4.966E-05 0.0 0.0 0.0 0.0 0.0 0.0	$\begin{array}{c} 3 \cdot 2 \cdot 94 = - 01 \\ 3 \cdot 226 = -01 \\ 2 \cdot 620 = -01 \\ 3 \cdot 275 = -02 \\ 3 \cdot 224 = -04 \\ 3 \cdot 04 \cdot 88 = -11 \\ 0 \cdot 0 \end{array}$	2.831E-03 2.801E-03 2.900E-03 2.772E-03 2.7707E-03 1.991E-03 1.036E-03 1.036E-03 1.001E-07 4.168E-18	1.697E-01 1.660E-01 1.340E-01 1.578E-02 1.347E-04 7.825E-12 0.0 0.0 0.0 0.0	$\begin{array}{c} 1.522E-07\\ 1.540E-07\\ 1.713E-07\\ 3.444E-07\\ 7.234E-07\\ 1.989E-05\\ 1.208E-05\\ 1.836E-05\\ 1.836E-05\\ 1.825E-05\\ 1.788E-05\\ \end{array}$	$\begin{array}{c} 2.138E-12\\ 2.418E-14\\ 9.948E-13\\ 4.623E-10\\ 8.521E-09\\ 1.391E-09\\ 6.817E-36\\ 2.382E-05\\ 5.374E-05\\ 5.789E-05\\ 1.511E-05\\ \end{array}$

Table B.10. Thermal power of LMFBR structural material waste as a function of decay time

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Appendix B.3. Toxicity of LMFBR Spent Fuel, High-Level Waste, and Structural Material Waste





	Toxicity (m <sup>3</sup> water/MTIHM)																	
Time - (years)	TOTAL	SR 90		PR143		PU239		R4226	PU240	AN241	PB210		RU106		TH229	CE144		RA225
1.000E-01 1.000E 00 1.000E 02 3.000E 02 1.000E 02 1.000E 04 3.000E 04 1.000E 04 1.000E 05 1.000E 05	9.630E 1 4.846E 1 2.146E 1 3.450E 1 2.3.450E 1 2.3.450E 1 2.3.450E 1 3.3.771E 0 3.1.159E 0 5.1.159E 0 5.2.049E 0 5.2.049E 0	3 2.428E 1 2.376E 1 1.918E 0 2.252E 9 1.927E 9 1.927E 9 0.0 8 0.0 8 0.0 8 0.0 8 0.0	11 11 10 08 01	9.469E 4.801E 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.	13 06	7.924E 7.924E 7.922E 7.902E 7.859E 7.709E 6.003E 3.402E 4.543E 1.068E	08 08 08 08 08 08 08 08 07 05	1.759E-02 4.423E-02 2.112E 00 1.033E 03 1.913E 04 3.128E 05 5.350E 07 1.2200E 08 1.262E 08 1.262E 08	1.453E 09 1.453E 09 1.454E 09 1.445E 09 1.415E 09 1.314E 09 5.059E 08 6.069E 07 3.629E 04 4.432E-01 4.407E-01	7.600E 08 1.058E 09 3.406E 09 6.821E 09 4.993E 09 1.625E 09 2.553E 05 4.978E 04 1.660E 02 1.441E-05 0.0	3.799E 3.766E 2.904E 1.499E 9.38AE 4.597E 1.604E 3.600E 3.785E 7.479E	000023 00023 00077 077 000	2.042E 1.100E 2.292E 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.	11 11 08	3.612E 3.615E 3.651E 5.990E 1.243E 2.275E 1.325E 5.177E 1.178E 1.259E	00 1.865E 1 00 8.367E 1 00 2.762E 0 01 0.0 03 0.0 05 0.0 05 0.0 06 0.0 06 0.0 07 0.0	11007	3.108E 00 2.892E 03 4.792E 00 3.917E 00 9.936E 00 1.820E 00 4.142E 00 9.104E 00 1.007E 0

Table B.11. Toxicity of LMFBR spent core fuel as a function of decay time

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Toxicity (m<sup>3</sup> water/MTIHM)

Tin	ne														
(yea	ers) .	CS137		RN 219		PU238	P0210	C\$134	NP237	PU242	AM243	1129	PU241	AC225	TH23 )
1.00 1.00 1.00 1.00 1.00 1.00 1.00 1.00	00E-000E0000E0000E0000E0000E0000E0000E	9.928   9.724   1.7.895   2.9.869   2.9.716   3.9.189   4.0.0   5.0.0   6.0.0   6.0.0	E 09 E 09 E 08 E 06 E 01	2.193E 2.768E 2.2940E 2.877E 2.035E 1.233E 7.862E 3.239E 4.587E 4.612E	00 01 02 05 05 06 06 06	3.543E 09 3.697E 09 3.502E 09 1.738E 09 3.721E 08 2.542E 06 1.774E-12 0.0 0.0 0.0	$\begin{array}{c} 3.616E-01\\ 5.029E-01\\ 4.151E-01\\ 2.141E 01\\ 6.302E 02\\ 1.340E 04\\ 6.567E 05\\ 2.292E 06\\ 5.143E 06\\ 5.408E 06\\ 1.068E 06\\ \end{array}$	2.264E 10 1.674E 10 8.122E 08 5.892E-05 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	6.858E 04 6.858E 04 7.798E 04 3.255E 05 1.74E 06 2.173E 06 2.161E 06 2.161E 06 1.980E 06 1.578E 06	4.600E 06 4.600E 06 54.601E 06 54.601E 06 54.601E 06 54.523E 06 54.523E 06 54.523E 06 53.850E 06 53.850E 06 53.850E 06	4.884E 07 4.883E 07 4.879E 07 4.838E 07 4.4858E 07 4.446E 07 1.909E 07 2.918E 06 4.049E 03 1.375E-01 1.333E-01	1.397E 06 1.403E 06 1.403E 06 1.403E 06 1.403E 06 1.403E 06 1.403E 06 1.403E 06 1.402E 06 1.397E 06 1.385E 06 1.385E 06	4.246E 09 4.066E 09 2.637E 09 3.465E 07 1.351E 04 1.060E 04 5.087E 03 9.953E 03 2.735E-07 0.0	$\begin{array}{c} 3.363E-01\\ 2.892E-01\\ 2.921E-01\\ 3.917E\\ 0917E\\ 0936E\\ 01\\ 1.820E\\ 04\\ 1.060E\\ 05\\ 4.142E\\ 05\\ 1.307E\\ 06\\ 1.307E\\ 06\\ $	7.432E-01 1.339E 00 1.77E 01 9.996E 02 5.862E 03 2.753E 04 2.960E 05 7.971E 05 1.792E 06 1.887E 06 3.740E 05

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	Toxicity (m <sup>3</sup> water/MTIHM)											
Time — (years)	TOTAL	SR 90	PU239 f	PR143	RA226	PU240	AM241	P8210	RU106	TH229	CE144	RA225
1.000E-01 1.000E 02 1.000E 02 3.000E 02 1.000E 03 1.000E 03 1.000E 04 3.000E 04 1.000E 04 1.000E 04 1.000E 04	1 .009E 14 5.016E 11 2.227E 11 2.3.560E 10 8.025E 05 1.268E 05 5.2.399E 06 5.2.399E 06 5.2.399E 06 5.2.399E 06 5.2.399E 06	2.524E 11 2.470E 11 1.994E 11 2.341E 10 2.005E 06 1.165E 01 0.0 3.0.0 3.0.0 3.0.0 3.0.0 7.0.0	9.282E 08 9.282E 08 9.282E 08 9.256E 08 9.205E 08 7.023E 08 3.975E 08 5.307E 07 1.678E 05 1.069E-01	9.919E 13 5.030E 06 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	1.762E-02 4.431E-02 2.118E 00 1.036E 03 1.918E 04 3.137E 05 1.538E 07 5.369E 07 1.207E 08 1.282E 08 2.849E 07	1.475E 09 1.475E 09 1.476E 09 1.466E 09 1.436E 09 5.134E 08 6.159E 07 3.682E 04 4.432E-01 4.407E-01	$\begin{array}{c} 7.603E 08\\ 1.059E 09\\ 3.411E 09\\ 6.831E 09\\ 5.001E 09\\ 1.628E 09\\ 2.553E 05\\ 4.978E 04\\ 1.660E 02\\ 1.441E-05\\ 0.0 \end{array}$	3.799E 00 3.767E 00 2.904E 00 1.503E 02 4.424E 03 9.410E 04 4.611E 06 1.6610E 07 3.620E 07 3.843E 07 8.543E 06	2.101E 11 1.131E 11 2.361E 08 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	3.614E 00 3.617E 00 6.104E 00 6.104E 00 4.995E 01 1.255E 03 2.286E 05 1.331E 06 5.200E 06 1.143E 07 1.264E 07	1.936E 11 8.687E 10 2.868E 07 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	3.109E 03 2.893E 00 2.923E 03 4.883E 00 3.997E 01 1.003E 03 1.828E 05 1.365E 36 4.161E 06 9.146E 06 1.312E 07

Table B.12. Toxicity of LMFBR spent core + axial blanket fuel as a function of decay time

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Toxicity (m<sup>3</sup> water/MTIHM)

(years)   RN219   CS137   PU238   P0210   CS134   NP237   PU242   AM243   I129   PU241   AC225   TH230     1.000E-01   2.258E   00   1.023E   10   3.554E   09   3.617E-01   2.283E   10   7.527E   04   4.601E   06   4.45E   06   4.253E   09   3.365E-01   7.44   3.36     1.000E   00   4.061E-01   1.001E   10   3.708E   09   5.030E-01   1.687E   10   7.566E   04   4.601E   06   4.883E   07   1.443E   06   4.602E   06   4.601E   06   4.883E   07   1.451E   06   2.640E   09   2.928-01   1.341     1.000E   02   1.917E   09   1.617E   08   4.452E   04   4.601E   06   4.883E   07   1.451E   06   2.640E   09   2.928-01   1.341     1.000E   02   1.617E   09   1.4743E	
1.000E-01 2.258E 00 1.023E 10 3.554E 09 3.617E-01 2.283E 10 7.527E 04 4.601E 06 4.884E 07 1.445E 06 4.253E 09 3.365E-01 7.445 1.000E 01 3.941E 00 8.136E 09 3.512E 09 4.151E-01 8.190E 08 8.469E 04 4.601E 06 4.883E 07 1.451E 06 4.072E 09 2.893E-01 1.341 1.000E 01 3.941E 00 8.136E 09 3.512E 09 4.151E-01 8.190E 08 8.469E 04 4.601E 06 4.879E 07 1.451E 06 2.640E 09 2.923E-01 1.776 1.000E 02 1.179E 02 1.017E 09 1.743E 09 2.147E 01 5.941E-05 3.325E 05 4.601E 06 4.878E 07 1.451E 06 3.470E 07 4.883E-01 1.002 3.000E 02 5.184E 02 1.001E 07 3.731E 08 6.320E 02 0.0 8.417E 05 4.601E 06 4.748E 07 1.451E 06 1.351E 04 3.997E 00 5.879 1.000E 03 3.052E 03 9.469E-01 2.546E 06 1.344E 04 0.0 1.750E 06 4.597E 06 4.446E 07 1.451E 06 1.060E 04 1.003E 02 2.761 1.000E 04 1.505E 05 0.0 1.774E-12 6.587E 05 0.0 2.1084E 06 4.523E 36 1.909E 07 1.451E 06 5.087E 03 1.828E 04 2.968 3.000E 04 9.350E 05 0.0 0.0 2.300E 06 0.0 2.100E 04 4.523E 36 1.909E 07 1.451E 30 05.087E 30 1.828E 04 2.968 3.000E 04 9.350E 05 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	0
1.000E 05 3.815E 06 0.0 0.0 5.171E 06 0.0 2.122E 06 3.850E 06 4.055E 03 1.445E 06 3.319E 00 4.161E 05 1.802 3.000E 05 5.395E 06 0.0 0.0 5.489E 06 0.0 1.989E 06 2.690E 06 1.375E-01 1.432E 06 2.735E-07 9.146E 05 1.916 1.000E 06 5.424E 06 0.0 0.0 1.221E 06 0.0 1.585E 06 7.677E 05 1.333E-01 1.389E 06 0.0 1.012E 06 4.274	SE-01 1E 00 '6E 01 '9E 03 '9E 03 '9E 04 6E 05 0E 05 2E 06 6E 06

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 T-61- 0 13	Taulaibu of IMES	D soort padia	l blankot fuel :	as a function of	decay time	

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			,		Toxi	city (m <sup>3</sup> wat	ter/MTIHM)					
Time - (years)	TOTAL	PU239	SR 90	PR143	RA226	PU240	RN219	PB210	RU106	CE144	CS137	AM24 1
$\begin{array}{c} 1.000E-01\\ 1.000E 00\\ 1.000E 02\\ 1.000E 02\\ 3.000E 02\\ 1.000E 03\\ 1.000E 04\\ 3.000E 04\\ 1.000E 04\\ 1.000E 05\\ 3.000E 05\\ 1.000E 06\end{array}$	1.705E 1 6.027E 1 2.843E 1 3.827E 0 5.627E 0 4.972E 0 3.506E 0 1.870E 0 2.980E 0 1.130E 0 1.130E 0	3 4.315E 0 4.315E 0 4.315E 9 4.314E 9 4.303E 8 4.278E 8 4.193E 8 3.235E 8 1.8193E 8 1.8192 7 2.421E 7 7.660E 7 1.342E	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	E 10 1.681E 1 E 10 8.523E 0 E 09 0.0 E 09 0.0 E 09 0.0 E 00 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	3 1.857E-04 5 4.111E-04 2.154E-02 1.154E-01 2.149E 02 3.507E 03 1.784E 05 7.017E 05 2.263E 06 5.737E 06 1.0001E 07	7.585E 07 7.585E 07 7.573E 07 7.504E 07 7.348E 07 2.626E 07 3.151E 06 1.883E 03 4.200E-06 3.020E-06	8.424E-01 1.131E 00 6.195E 00 2.6.305E 02 8.262E 04 6.335E 02 8.262E 04 6.335E 05 1.812E 06 2.554E 06 5.2557E 06	5.176E-03 5.212E-03 4.499E-03 1.669E 00 4.954E 01 1.051E 03 5.350E 04 2.105E 05 6.786E 05 1.720E 06 3.001E 06	2.138E 10 1.151E 10 2.394E 07 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	2.517E 1.129E 3.729E 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.	10 1.091E 09 10 1.058E 09 06 8.679E 08 1.084E 08 1.010E-01 0.0 0.0 0.0 0.0 0.0 0.0 0.0	1.616E 06 3.415E 06 1.757E 07 3.848E 07 9.171E 06 5.280E 00 6.537E-02 2.167E-04 1.879E-11 0.0

Toxicity (m<sup>3</sup> water/MTIHM)

Time (years)	P0210	I 129	TH229	C\$134	RA225	SR 89	TH230	RA223	PU238	PA231	PM147	NB 95
1.000E-0 1.000E 0 1.000E 0 3.000E 0 1.000E 0 1.000E 0 3.000E 0 3.000E 0 1.000E 0 1.000E 0 1.000E 0	4.183E-04 6.797E-04 6.433E-04 2.384E-01 2384E-01 2384E-01 5.1.503E 02 4.7.643E 03 4.3.007E 04 5.9.692E 04 5.2.458E 05 5.4.287E 05	1.705E 05 1.715E 05 1.715E 05 1.715E 05 1.715E 05 1.715E 05 1.714E 05 1.714E 05 1.714E 05 1.692E 05 1.641E 05	1.600E-02 1.630E-02 2.27E-02 4.267E-01 3.655E 00 4.309E 01 4.006E 03 2.234E 04 8.616E 04 1.889E 05 2.088E 05	1.036E 09 7.654E 08 3.715E 07 2.695E-06 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	$\begin{array}{c} 1.391E-02\\ 1.304E-02\\ 1.818E-02\\ 3.413E-01\\ 2.923E\ 00\\ 3.448E\ 01\\ 3.204E\ 03\\ 1.787E\ 04\\ 6.890E\ 04\\ 1.511E\ 05\\ 1.670E\ 05\\ \end{array}$	4.718E 10 5.177E 08 1.312E-11 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	$\begin{array}{c} 6.056E-03\\ 1.167E-02\\ 1.866E-01\\ 1.121E\ 01\\ 6.583E\ 01\\ 3.085E\ 02\\ 3.476E\ 03\\ 1.047E\ 04\\ 8.592E\ 04\\ 1.501E\ 05\\ \end{array}$	3.611E-02 4.850E-02 6.55E-01 6.896E 00 2.703E 01 1.218E 03 3.542E 03 1.985E 04 1.995E 05 1.096E 05	4.283E 07 4.262E 07 3.972E 07 1.951E 07 4.023E 06 1.662E 04 4.523E-16 0.0 0.0 0.0	3.175E-01 3.613E-01 1.019E 00 7.608E 00 2.364E 01 9.467E 01 2.754E 03 1.544E 04 6.039E 04 8.477E 04 8.523E 04	3.388E 08 2.446E 08 2.269E 07 1.068E-03 3.3 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	4.643E 09 2.134E 08 7.452E-08 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0

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Table B.14. Toxicity of LMFBR high-level waste as a function of decay time

					Toxic	ity (m <sup>3</sup> wate	r/MTIHM)					
Time (years)	TOTAL	SR 90	RA226 A	M241	AM243	PR143	PU239	TH229	RU136	RA225	PB210	CE144
3.500E-01 1.250E 00 1.025E 00 1.003E 01 3.003E 01 1.000E 01 3.000E 00 1.000E 00 1.000E 00 1.000E 00	1 1.219E 12 3.028E 11 1 1.432E 11 2 1.719E 10 2 5.588E 08 3 1.531E 08 4 2.522E 07 4 1.276E 07 5 1.049E 07 5 1.049E 07 5 3.906E 06	1.711E 11 1.675E 11 1.352E 11 1.357E 10 1.359E 08 7.893E 00 0.0 0.0 0.0 0.0 0.0	$\begin{array}{c} 1.333E-02 & 5.\\ 1.343E-02 & 5.\\ 5.161E-02 & 5.\\ 3.462E & 01 & 4.\\ 7.144E & 02 & 3.\\ 1.323E & 04 & 1.\\ 7.142E & 05 & 1.\\ 2.505E & 06 & 3.\\ 5.617E & 06 & 1.\\ 5.846E & 06 & 9.\\ 9.897E & 05 & 0.\\ \end{array}$	.127E 08 .129E 08 .130E 08 .563E 08 .313E 08 .081E 08 .609E 05 .147E 04 .043E 02 .053E-06	3.088E 07 3.087E 07 3.085E 07 3.059E 07 3.002E 07 2.811E 07 1.207E 07 1.845E 06 2.570E 06 8.694E-02 8.694E-02	6.938E 11 3.517E 04 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	3.729E 06 3.730E 06 3.741E 06 3.812E 06 3.931E 06 4.317E 06 6.735E 06 5.534E 06 8.228E 05 2.604E 03 6.741E-02	$\begin{array}{c} 1 \cdot 146E - 02\\ 1 \cdot 157E - 02\\ 2 \cdot 070E - 02\\ 1 \cdot 001E & 00\\ 1 \cdot 066E & 01\\ 1 \cdot 661E & 02\\ 2 \cdot 153E & 04\\ 1 \cdot 234E & 05\\ 4 \cdot 810E & 05\\ 1 \cdot 057E & 06\\ 1 \cdot 169E & 06\\ \end{array}$	1.188E 11 6.398E 10 1.307E 08 0.3 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	3.409E-01 9.255E-03 1.656E-02 8.012E-01 8.529E 00 1.328E 02 1.722E 04 9.875E 04 3.848E 05 9.352E 05	2.425E 00 2.358E 00 1.764E 00 5.033E 00 1.633E 02 3.967E 03 2.142E 05 7.512E 05 1.684E 05 1.753E 06 2.968E 05	1.057E 11 4.744E 10 1.566E 07 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0

Toxicity (m<sup>3</sup> water/MTIHM)

Time <sup>—</sup> (years)	PU240	CS137	NP237	SN126	CS134	SR 89	PD107	P0210	AC225	NP 2 39	RN219	SE 79
3.500E-01 1.250E 00 1.025E 01 1.003E 02 3.003E 02 1.000E 03 1.000E 04 3.000E 04 3.000E 05 3.000E 05	4.820E 06 4.974E 06 6.248E 06 9.181E 06 9.086E 06 8.436E 06 3.249E 06 3.897E 05 2.330E 02 1.506E-03	6.830E 09 6.689E 09 5.433E 09 6.791E 08 6.682E 06 6.319E-01 0.0 0.0 0.0	5.663E 04 5.663E 04 5.883E 04 7.779E 04 1.115E 05 1.717E 05 2.011E 05 2.005E 05 1.962E 05 1.839E 05	6.853E 05 6.853E 05 6.852E 05 6.848E 05 6.839E 05 6.805E 05 6.394E 05 5.566E 05 3.427E 05 8.567E 04	1.364E 10 1.008E 10 4.893E 08 3.550E-05 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	5.110E 10 5.612E 08 1.422E-11 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	1.037E 05 1.037E 05 1.037E 05 1.037E 05 1.037E 05 1.037E 05 1.037E 05 1.034E 05 1.024E 05 1.004E 05	2.701E-01 3.233E-01 2.550E-01 7.190E-01 2.333E 01 5.667E 02 3.060E 04 1.073E 05 2.506E 05 2.504E 05	7.295E-02 9.256E-04 1.656E-03 8.012E-02 8.529E-01 1.328E 01 1.722E 03 9.875E 03 3.848E 04 8.458E 04	1.235E 06 1.235E 06 1.235E 06 1.223E 06 1.223E 06 1.124E 06 4.828E 05 7.379E 04 1.030E 02 3.477E-03	3.354E-01 7.196E-01 2.480E 00 8.057E 00 1.046E 01 2.284E 01 9.269E 02 7.9992E 03 4.289E 04 6.448E 04	1.670E 05 1.6670E 05 1.669E 05 1.664E 05 1.664E 05 1.652E 05 1.501E 05 1.5212E 05 5.744E 04 6.798E 03
1.000E 06	1.593E-03	0.0	1.466E 05	6.690E 02	0.0	0.0	9.320E 04	4.2405 04	9.352E 04	3.370E-03	6.491E 04	3.878E 00

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Fig. B.15. Toxicity of LMFBR structural material waste as a function of decay time.

Table B.15. Toxicity of LMFBR structural material waste as a function of decay time

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Toxicity	(m <sup>3</sup>	water/MTIHM)

Time (years) 3.500E-01 1.250E 00 1.025E 01 1.003E 02 3.003E 02 1.000E 03	TOTAL 2.201E 09 5.306E 08 1.033E 08 1.576E 07 3.625E 06 1.499E 06	SR 90 8.559E 07 8.378E 07 6.762E 07 7.938E 06 6.799E 04 3.948E-03	PU239 3.731E 05 3.731E 05 3.730E 05 3.721E 05 3.700E 05 3.700E 05 3.628E 05	RA226 7.452E-06 1.734E-05 7.144E-04 3.320E-01 6.120E 00 9.992E 01	MN 54 5.245E 08 2.530E 08 1.724E 05 0.0 0.0 0.0	PU240 4.805E 05 4.805E 05 4.807E 05 4.677E 05 4.677E 05 4.677E 05 4.342E 05	CO 58 9.602E 08 3.839E 07 4.004E-07 0.0 0.0	AM241 2.672E 05 3.609E 05 1.098E 06 2.167E 06 1.586E 06 5.162E 05	CO 60 7.176E 07 6.375E 07 1.951E 07 1.410E 02 5.300E-10 0.0	NI 63 7.783E 06 7.730E 06 7.223E 06 3.666E 06 8.125E 05 4.168E 03	PB210 1.216E-03 1.183E-03 9.139E-04 4.823E-02 1.411E 00 2.996E 01	N3 94 9.163E 04 9.163E 04 9.160E 04 9.132E 04 9.070E 04 8.855E 04
1.000E 04 3.000E 04 1.000E 05 3.000E 05 1.000E 06	5.553E 05 2.534E 05 9.097E 04 6.692E 04 2.567E 04		2.817E 05 1.592E 05 2.124E 04 6.756E 01 3.384E-05	1.711E 04 3.860E 04 4.158E 04 1.085E 04	0.0 0.0 0.0 0.0	2.006E 04 1.199E 01 1.402E-04 1.394E-04	0.0 0.0 0.0	1.574E 01 5.220E-02 4.531E-09 0.0	0.0	0.0 0.0 0.0 0.0	5.130E 03 1.157E 04 1.247E 04 3.253E 03	3.290E 04 3.014E 03 3.276E 00 1.364E-10
:					Toxi	city (m <sup>3</sup> wat	er/MTIHM)					
Time — (years)	TH229	PR143	RA225	NI 59	RN219	RU106	CE144	NO 93	CS137	PU238	FE 55	P0210
3.500 E-01 1.250E 00 1.025E 01 1.003E 02 3.003E 02 1.000E 03 1.000E 04 1.000E 05 3.000E 05 1.000E 05	1.146E-03 1.147E-03 1.160E-03 2.014E-03 1.651E-02 4.049E-01 7.302E 01 4.252E 02 1.661E 03 3.651E 03 4.037E 03	3.471E 00 1.759E 01 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	9.179E-04 9.283E-04 1.612E-03 1.321E-02 3.239E-01 5.842E 01 3.402E 02 1.328E 03 2.921E 03 3.229E 03	1.619E 04 1.619E 04 1.619E 04 1.619E 04 1.618E 04 1.618E 04 1.605E 04 1.485E 04 1.249E 04 6.808E 03 1.203E 03 2.803E 00	2.769E-04 3.736E-04 2.471E-03 6.705E-02 2.800E+01 1.487E00 6.280E01 3.810E02 1.540E03 2.175E03 2.166E03	5.943E 07 3.201E 07 6.662E 04 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	5.289E 07 2.373E 07 7.636E 03 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	6.871E 04 6.869E 04 6.857E 04 6.736E 04 6.474E 04 5.636E 04 9.474E 03 1.801E 02 1.706E-04 1.053E-21 0.0	3.416E 06 3.346E 06 2.718E 06 3.397E 05 3.343E 03 3.162E-04 0.0 0.0 0.0 0.0	1.154E 06 1.184E 06 1.116E 05 5.540E 05 1.185E 05 8.040E C2 5.603E-16 0.0 0.0 0.0	1.448E 07 1.139E 07 1.034E 06 3.928E-05 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	1.351E-04 1.622E-04 1.306E-03 2.016E-01 4.280E 00 2.097E 02 7.329E 02 1.653E 03 1.781E 33 4.648E 02

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Appendix C. CHARACTERISTICS OF FFTF SPENT FUEL, HIGH-LEVEL WASTE, AND STRUCTURAL MATERIAL WASTE Appendix C.l. Radioactivity of FFTF Spent Fuel, High-Level Waste, and Structural Material Waste



Fig. C.l. Radioactivity of FFTF spent fuel as a function of decay time.

	Radioactivity (Ci/MTIHM)											
Time - (years)	TOTAL	PU239	C\$137	PU240	PU241	AM241	SR 90	RH106	RU106	PR144	CE144	NB 95
$\begin{array}{c} 1.000E-01\\ 3.000E-01\\ 5.000E-01\\ 0.000E 00\\ 5.000E 00\\ 5.000E 00\\ 5.000E 01\\ 1.000E 01\\ 1.000E 01\\ 1.000E 02\\ 1.500E 02\\ 3.000E 02\\ 1.000E 03\\ 1.000E 03\\ 1.000E 04\\ 1.000E 05\\ 1.000E 05\\ 1.000E 05\\ 1.000E 06\\ 0.000E 05\\ 1.000E 06\\ 0.000E 05\\ 0.00$	3.849E 07 1.867E 07 7.937E 08 6.069E 06 3.114E 06 6.714E 05 4.599E 09 2.079E 05 8.708E 04 5.476E 04 3.575E 04 2.996E 04 1.707E 04 1.186E 03 5.476E 04 1.707E	1.210E 1.210E 1.210E 1.210E 1.210E 1.210E 1.210E 1.210E 1.209E 1.209E 1.209E 1.209E 1.209E 1.209E 1.192E 1.192E 1.192E 2.149E 3.6789E 3.819E-	$\begin{array}{c} 4 & 2 & 721E & 05 \\ 4 & 2 & 709E & 05 \\ 4 & 2 & 696E & 05 \\ 4 & 2 & 663E & 05 \\ 4 & 2 & 663E & 05 \\ 4 & 2 & 663E & 05 \\ 4 & 2 & 604E & 05 \\ 4 & 2 & 605E & 05 \\ 4 & 2 & 635E & 05 \\ 4 & 2 & 635E & 04 \\ 4 & 2 & 663E & 02 \\ 4 & 2 & 623E & 00 \\ 3 & 0 & 0 & 0 \\$	7.876E 03 7.876E 03 7.876E 03 7.873E 03 7.873E 03 7.873E 03 7.873E 03 7.873E 03 7.870E 03 7.858E 03 7.858E 03 7.858E 03 7.750E 03 7.750E 03 7.729E 03 2.728E 03 3.271E 02 1.956E-08 2.977E-08	$\begin{array}{c} 4.277E \ 05\\ 4.235E \ 05\\ 4.196E \ 05\\ 4.196E \ 05\\ 3.376E \ 05\\ 3.376E \ 05\\ 3.376E \ 05\\ 3.670E \ 04\\ 3.488E \ 03\\ 3.142E \ 05\\ 3.670E \ 04\\ 3.488E \ 03\\ 3.142E \ 01\\ 5.3870E \ 04\\ 3.488E \ 03\\ 3.142E \ 01\\ 5.343E \ 03\\ 3.142E \ 01\\ 4.804E \ 03\\ 4.804E \ 0$	$\begin{array}{c} \textbf{7.446E} & \textbf{03}\\ \textbf{7.581E} & \textbf{03}\\ \textbf{7.714E} & \textbf{03}\\ \textbf{7.714E} & \textbf{03}\\ \textbf{7.876E} & \textbf{03}\\ \textbf{8.669E} & \textbf{03}\\ \textbf{1.037E} & \textbf{04}\\ \textbf{1.583E} & \textbf{04}\\ \textbf{1.583E} & \textbf{04}\\ \textbf{1.914E} & \textbf{04}\\ \textbf{1.914E} & \textbf{04}\\ \textbf{1.914E} & \textbf{04}\\ \textbf{1.371E} & \textbf{04}\\ \textbf{9.952E} & \textbf{03}\\ \textbf{1.806E} & \textbf{02}\\ \textbf{4.861E-03}\\ \textbf{1.806E-04}\\ \textbf{1.604E-04}\\ \textbf{1.604E-04}\\ \textbf{1.604E-03}\\ \textbf{0.93E-13}\\ \textbf{0.0} \end{array}$	$\begin{array}{c} 1.051E 05\\ 1.046E 05\\ 1.046E 05\\ 1.036E 05\\ 1.029E 05\\ 1.005E 05\\ 1.005E 05\\ 3.58E 04\\ 8.307E 04\\ 8.307E 04\\ 3.206E 04\\ 9.753E 03\\ 2.967E 03\\ 8.352E 01\\ 7.147E-01\\ 4.848E-06\\ 1.026E-26\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.$	$\begin{array}{c} 2.055 \\ 0.55 \\ 1.561 \\ 0.561 \\ 0.561 \\ 0.563 \\ 0.563 \\ 0.5563 \\ 0.5563 \\ 0.5563 \\ 0.57$	$\begin{array}{c} 2. \ \ \ \ \ \ \ \ \ \ \ \ \ \ \ \ \ \ $	2.113E 06 1.768E 06 1.479E 06 1.184E 06 3.892E 05 3.892E 05 3.892E 02 4.244E-02 1.056E-13 4.828E-33 2.207E-52 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.	2.113E 06 1.758E 06 1.479E 06 1.479E 05 3.688E 05 3.688E 04 3.132E 02 4.2244E-02 1.0566-13 4.828E-33 2.207E-52 0.3 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	4.840E 06 2.929E 06 1.502E 06 5.979E 05 2.288E 03 3.1297E-02 7.226E-02 7.02 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0

### Table C.1. Radioactivity of FFTF spent fuel as a function of decay time

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Radioactivity (Ci/MTIHM)

Time (years)	PM147	ZR 95	RU103	RH103M	CE141	NP237	PA233	PU238	U233	TH229	AC225	RA225
$\begin{array}{c} 1 \cdot 000 = -01\\ 3 \cdot 000 = -01\\ 5 \cdot 000 = -01\\ 4 \cdot 000 = 00\\ 2 \cdot 000 = 00\\ 2 \cdot 000 = 00\\ 2 \cdot 000 = 00\\ 1 \cdot 000 = 01\\ 2 \cdot 000 = 01\\ 1 \cdot 000 = 01\\ 1 \cdot 000 = 02\\ 1 \cdot 000 = 02\\ 1 \cdot 000 = 03\\ 3 \cdot 000 = 03\\ 1 \cdot 000 = 04\\ 1 \cdot 000 = 05\\ 3 \cdot 000 = 05\\ 1 \cdot 000 = 05\\ 0 \cdot 00 = 05\\ 0$	$\begin{array}{c} 4.367 \pm 05\\ 4.169 \pm 05\\ 3.955 \pm 05\\ 3.702 \pm 05\\ 2.661 \pm 05\\ 2.661 \pm 05\\ 3.214 \pm 05\\ 3.214 \pm 05\\ 4.2289 \pm 03\\ 8.265 \pm -01\\ 2.5772 \pm -12\\ 1.702 \pm -29\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.$	$\begin{array}{c} \textbf{3.711E} & \textbf{06} \\ \textbf{1.682E} & \textbf{06} \\ \textbf{7.623E} & \textbf{05} \\ \textbf{2.835E} & \textbf{05} \\ \textbf{2.835E} & \textbf{05} \\ \textbf{1.054E} & \textbf{05} \\ \textbf{2.016E} & \textbf{03} \\ \textbf{1.409E-02} \\ \textbf{3.602E-11} \\ \textbf{2.354E-28} \\ \textbf{0.0} \\ $	4.093E 06 1.128E 06 3.108E 05 6.202E 04 1.238E 04 2.338E 04 8.018E-08 8.111E-22 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	3.690E 06 1.017E 06 2.801E 05 5.593E 04 1.116E 01 7.226E-08 7.313E-22 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	2.992E 06 6.301E 05 1.328E 05 1.895E 03 1.123E 00 8.027E-11 9.885E-28 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.	$\begin{array}{c} 1.510E-01\\ 1.516E-01\\ 1.520E-01\\ 1.527E-01\\ 1.533E-01\\ 1.561E-01\\ 1.6653E-01\\ 1.6653E-01\\ 1.841E-01\\ 2.307E-01\\ 4.054E-01\\ 1.841E-01\\ 2.307E-01\\ 1.842E-01\\ 1.842E-00\\ 1.763E-00\\ 1.762E-00\\ 1.$	$\begin{array}{c} 1.419 \pm -01\\ 1.499 \pm -01\\ 1.516 \pm 01\\ 1.551 \pm -01\\ 1.551 \pm -01\\ 1.561 \pm -01\\ 1.561 \pm -01\\ 1.665 \pm -01\\ 1.665 \pm -01\\ 1.645 \pm -01\\ 1.655 \pm 00\\ 1.763 \pm 00\\ 2.523 \pm 00\\ 3.629 \pm 00\\ 3.629 \pm 00\\ 4.491 \pm 00\\ 4.318 \pm 00\\ 3.280 \pm 00\\ \end{array}$	7.202E 03 7.663E 03 8.289E 03 8.479E 03 8.479E 03 8.4762E 03 8.4762E 03 8.37E 03 8.337E 03 6.184E 03 2.957E 03 1.0092E 02 1.392E 01 1.2062E 02 1.392E 01 1.2062E 01 0.0 0.0 0.0	$\begin{array}{c} 2,595 \pm -05\\ 2,608 \pm -05\\ 2,621 \pm -05\\ 2,638 \pm -05\\ 2,729 \pm -05\\ 2,729 \pm -05\\ 3,328 \pm -05$	2.013E-06 2.014E-06 2.014E-06 2.015E-06 2.015E-06 2.025E-06 2.025E-06 2.025E-06 2.025E-06 2.0240E-06 2.0240E-06 4.265E-05 5.5666E-05 5.5524E-05 5.5524E-03 5.226E-03 6.187E-02 3.654E-01 1.433E 00 3.154E 00 3.488E	$\begin{array}{c} 2.46E-0.6\\ 2.023E-0.6\\ 2.014E-0.6\\ 2.0114E-0.6\\ 2.0118E-0.6\\ 2.0125E-0.6\\ 2.025E-0.6\\ 2.025E-0.6\\ 2.025E-0.6\\ 2.025E-0.6\\ 2.025E-0.6\\ 2.025E-0.6\\ 1.55654E-0.5\\ 3.5226E-0.5\\ 3.524E-0.6\\ 1.55654E-0.5\\ 3.5254E-0.6\\ 1.55654E-0.5\\ 3.5254E-0.6\\ 1.55654E-0.5\\ 3.5254E-0.5\\ 3.5254E-0.5\\ 3.5254E-0.5\\ 3.5254E-0.5\\ 3.5254E-0.5\\ 3.5254E-0.5\\ 3.5554E-0.5\\ 3.5254E-0.5\\ 3.5554E-0.5\\ 3.555$	$\begin{array}{c} 2.119E-06\\ 2.017E-06\\ 2.015E-06\\ 2.015E-06\\ 2.015E-06\\ 2.015E-06\\ 2.025E-06\\ 2.039E-06\\ 2.039E-06\\ 2.039E-06\\ 2.073E-06\\ 2.240E-06\\ 4.265E-06\\ 4.265E-06\\ 4.265E-05\\ 3.524E-04\\ 3.565E-01\\ 1.565E-03\\ 3.524E-03\\ 3.154E\\ 00\\ 3.154E\\ 00\\ 3.488E\\ 00\\ \end{array}$

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Fig. C.2. Radioactivity of FFTF high-level waste as a function of decay time.

		· · · ·	• . •		Radi	oactivity (C	i/MTIHM)					
Time (years)	TOTAL	C\$137	AM241	PU239	TC 99	SR 90	RH106	RU106	PR144	CE144	ZR 93	PH147
$\begin{array}{c} 5.380E-01\\ 7.380E-01\\ 1.238E&00\\ 1.438E&00\\ 2.438E&00\\ 5.438E&00\\ 5.438E&00\\ 1.044E&01\\ 2.044E&01\\ 1.004E&02\\ 1.504E&02\\ 1.504E&02\\ 1.504E&02\\ 1.504E&02\\ 1.004E&02\\ 1.000E&03\\ 1.000E&03\\ 1.000E&03\\ 1.000E&03\\ 1.000E&03\\ 1.000E&05\\ 1.000E&05\\ 1.000E&06\\ \end{array}$	$\begin{array}{c} 1.008E \\ 0\\ 7.372E \\ 0\\ 0\\ 843E \\ 0\\ 2.053E \\ 0\\ 2.053E \\ 0\\ 0\\ 2.557E \\ 0\\ 0\\ 1.306E \\ 0\\ 0\\ 1.305E \\ 0\\ 0\\ 1.305E \\ 0\\ 0\\ 1.305E \\ 0\\ 0\\ 1.305E \\ 0\\ 0\\ 0\\ 1.982E \\ 0\\ 0\\ 0\\ 1.982E \\ 0\\ 0\\ 0\\ 0\\ 1.982E \\ 0\\ 0\\ 0\\ 0\\ 0\\ 0\\ 0\\ 0\\ 0\\ 0\\ 0\\ 0\\ 0$	7 2.701E 0 5 2.688E 0 6 2.665E 0 6 2.665E 0 6 2.645E 0 5 2.411E 0 5 2.418E 0 5 2.148E 0 5 2.148E 0 5 2.148E 0 5 2.148E 0 4 2.685E 0 4 2.685E 0 4 2.685E 0 4 2.685E 0 3 2.499E 0 3 2.499E 0 3 2.499E 0 3 2.499E 0 1 0.0 1 0.0 1 0.0	57.691E03 57.689E03 57.689E03 57.689E03 57.689E03 57.689E03 57.692E03 57.692E03 57.592E03 57.492E03 57.	$\begin{array}{c} 6.065E & 01\\ 6.052E & 01\\ 6.052E & 01\\ 5.92E & 01\\ 5.92E & 01\\ 5.92E & 01\\ 5.624E & 01\\ 2.676E & 01\\ 3.588E & 01\\ 1.133E-02\\ 9.197E-11\end{array}$	$\begin{array}{c} 1.851E & 01\\ 1.850E & 01\\ 1.850E & 01\\ 1.850E & 01\\ 1.848E & 01\\ 1.888E & 01$	$\begin{array}{c} 1.043E 05\\ 1.038E 05\\ 1.038E 05\\ 1.027E 05\\ 9.971E 04\\ 9.284E 04\\ 6.497E 04\\ 3.181E 04\\ 9.677E 03\\ 2.944E 03\\ 8.285E 01\\ 7.103E-01\\ 4.8185E 01\\ 7.103E-01\\ 4.810E-01\\ 0.00E-26\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.$	$\begin{array}{c} 1.524E 06\\ 1.328E 06\\ 9.749E 05\\ 6.749E 05\\ 5.245E 03\\ 1.685E 03\\ 1.756E 00\\ 2.256E-24\\ 2.638E-39\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.$	1.524E 06 1.328E 06 9.749E 05 8.209E 05 4.127E 05 5.245E 06 1.685E 03 1.756E-09 2.256E-24 2.638E-39 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.	1.434E 06 1.200E 06 8.037E 05 6.433E 05 1.8825E 04 2.125E 02 2.880E-02 7.168E-14 3.277E-33 1.498E-52 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.	1.434E 06 1.200E 06 8.0037E 05 6.433E 05 2.640E 05 1.825E 02 2.880E-02 2.880E-02 7.168E-14 3.277E-33 1.498E-52 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.	$\begin{array}{c} 1.851E \\ 0.01\\ 0.867E \\ 0.02\\ 0.883E \\ 0.01\\ 0.902E \\ 0.02\\ 0.1992E \\ 0.02\\ 0.199E \\ 0.02$	3.926E 05 3.724E 05 3.532E 05 3.306E 05 2.376E 05 2.376E 05 2.376E 05 2.870E 04 2.044E 03 7.381E-01 1.352E-06 2.475E-12 1.520E-29 0.0
• •	•	· .										
,		-			Radi	oactivity (C	i/MTIHM)					
Time (years)	PU240	CS135	NB 95	NP237	PA233	U233	TH229	AC225	RA225	FR221	AT217	BI213
5.380E-01 7.380E-01 9.380E-01 1.238E00 1.438E00 2.438E00 1.438E00 2.438E00 1.044E01 2.004E02 2.004E02 2.000E03 2.000E05 2	$\begin{array}{c} 3.948E \\ 0.3.948E \\ 0.3.948E \\ 0.3.948E \\ 0.3.948E \\ 0.3.954E \\ 0.3.955E \\ 0.3.955E \\ 0.3.955E \\ 0.3.955E \\ 0.3.754E \\ 0.3.955E \\ 0.3.754E \\ 0.3.955E \\ 0.3.754E \\ 0.3.955E \\ 0.3.754E \\ 0.3.75$	1 2.137E 0 1 2.136E 0 1 2.13	$\begin{array}{c} 0 & 1.314E & 06\\ 0 & 6.270E & 05\\ 0 & 2.910E & 05\\ 0 & 1.01E & 05\\ 0 & 4.122E & 04\\ 0 & 8.203E & 02\\ 0 & 5.543E - 03\\ 0 & 1.6477E - 11\\ 0 & 9.258E - 29\\ 0 & 0.0 & 0\\ 0 & 0 & 0\\ 0 & 0 & 0\\ 0 & 0 & 0\\ 0 & 0 &$	$\begin{array}{c} 1.525E-01\\ 1.535E-01\\ 1.535E-01\\ 1.548E-01\\ 1.548E-01\\ 1.573E-01\\ 1.647E-01\\ 1.771E-01\\ 2.015E-01\\ 2.015E-01\\ 2.015E-01\\ 3.842E-01\\ 4.871E-01\\ 1.842E-01\\ 1.842E-01\\ 1.847E-01\\ 1.847E-01\\ 1.847E-01\\ 1.847E-01\\ 1.847E-01\\ 1.847E-01\\ 1.847E-01\\ 1.847E-01\\ 1.848E-01\\ 1.848E-01\\ 1.888E-01\\ 1.88$	$\begin{array}{c} 1.521E-01\\ 1.528E-01\\ 1.533E-01\\ 1.5539E-01\\ 1.5539E-01\\ 1.573E-01\\ 1.577E-01\\ 2.015E-01\\ 2.015E-01\\ 2.015E-01\\ 3.842E-01\\ 3.842E-01\\ 1.017E-00\\ 1.607E-00\\ 1.707E-00\\ 1.706E-00\\ 1.706E-00\\ 1.706E-00\\ 1.561E-00\\ 1.561E-00\\ 1.244E-00\\ 1.$	$\begin{array}{c} 1.976E-07\\ 3.310E-07\\ 4.648E-07\\ 6.327E-07\\ 8.013E-07\\ 1.5565E-06\\ 3.741E-06\\ 1.575E-05\\ 4.686E-05\\ 1.88E-04\\ 2.141E-04\\ 6.233E-04\\ 1.401E-03\\ 4.098E-03\\ 1.821E-02\\ 6.942E-02\\ 2.068E-01\\ 1.185E\\ 00\\ 1.321E\\ 00\\ \end{array}$	$\begin{array}{c} 2.019 \pm -06\\ 2.030 \pm -06\\ 2.030 \pm -06\\ 2.030 \pm -06\\ 2.48 \pm -06\\ 3.24 6 \pm -06\\ 3.24 6 \pm -06\\ 2.73 \pm -05\\ 1.502 \pm -04\\ 2.052 \pm -04$	2.020E-06 2.019E-06 2.019E-06 2.019E-06 2.019E-06 2.019E-06 2.021E-06 2.030E-06 2.030E-06 2.482E-06 3.246E-06 3.246E-06 3.246E-04 2.358E-02 1.502E-04 2.358E-02 1.390E-01 1.497E00 1.323E00	$\begin{array}{c} 2. \ \ \ \ \ \ \ \ \ \ \ \ \ \ \ \ \ \ $	2.020E-06 2.019E-06 2.019E-06 2.019E-06 2.019E-06 2.021E-06 2.021E-06 2.030E-06 2.111E-06 3.246E-06 3.246E-06 3.246E-06 3.246E-03 1.502E-04 2.368E-02 1.390E-01 1.197E 00 1.323E 00	$\begin{array}{c} 2.023E-06\\ 2.019E-06\\ 2.019E-06\\ 2.019E-06\\ 2.019E-06\\ 2.019E-06\\ 2.019E-06\\ 2.019E-06\\ 2.032E-06\\ 2.032E-06\\ 3.246E-06\\ 3.246E-06\\ 3.246E-06\\ 2.734E-05\\ 1.502E-04\\ 2.368E-02\\ 1.390E-01\\ 1.390E-01\\ 1.197E 00\\ 1.323E 00\\ \end{array}$	2.020E-06 2.019E-06 2.019E-06 2.019E-06 2.019E-06 2.019E-06 2.021E-06 2.021E-06 2.030E-06 2.030E-06 2.482E-06 3.246E-06 3.236E-06 3.246E-06 3.246E-06 3.236E-06 3.236E-06 3.236E-06 3.246E-06 3.236E-00 3.236E-00 3.256E-00 3.256E-00 3.256E-00 3.256E

Table C.2. Radioactivity of FFTF high-level waste as a function of decay time

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Fig. C.3. Radioactivity of FFTF structural material waste as a function of decay time.

Table C.3.	Radioactivity	of FFTF	structural	material	waste as	a	function	of	decay tim	ne
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Radioactivity (Ci/MTIHM)											
Time — (years)	TOTAL	NI 59	PU239 NI 63	CO 58	MN 54	CD 60	PU240 FE 55	AM241 C 14	TC 99		
$\begin{array}{c} 5.380 \pm -01\\ 7.380 \pm -01\\ 9.380 \pm -01\\ 1.238 \pm 00\\ 1.438 \pm 00\\ 2.438 \pm 00\\ 5.438 \pm 00\\ 5.438 \pm 00\\ 1.044 \pm 01\\ 2.044 \pm 01\\ 1.004 \pm 02\\ 1.504 \pm 02\\ 1.504 \pm 02\\ 1.504 \pm 02\\ 1.000 \pm 02\\ 1.000 \pm 03\\ 0.00 $	$\begin{array}{c} 4.445E \ 05\\ 2.610E \ 05\\ 1.656E \ 05\\ 1.050E \ 05\\ 3.160E \ 04\\ 3.160E \ 04\\ 3.160E \ 04\\ 3.13E \ 03\\ 2.313E \ 03\\ 3.943E \ 02\\ 2.413E \ 02\\ 1.645E \ 01\\ 1.651E \ 01\\ 1.651E \ 01\\ 1.6538E \ 01\\ 5.145E \ 00\\ 1.638E \ 00\\ 2.797E-01\\ 3.153E-02 \end{array}$	2.865E 00 2.865E 00 2.865E 00 2.865E 00 2.865E 00 2.865E 00 2.865E 00 2.865E 00 2.864E	$\begin{array}{c} 6.068E 00 4.0966\\ 6.068E 00 4.0966\\ 6.068E 00 4.0836\\ 6.067E 00 4.0836\\ 6.067E 00 4.0386\\ 6.067E 00 4.0386\\ 6.067E 00 3.9477\\ 6.066E 00 3.9477\\ 6.066E 00 3.8016\\ 6.059E 00 2.8126\\ 6.059E 00 2.8126\\ 6.059E 00 1.3246\\ 6.059E 00 1.3246\\ 6.059E 00 2.8126\\ 5.895E 00 2.1916\\ 5.895E 00 2.1916\\ 5.895E 00 2.1916\\ 5.895E 00 2.1916\\ 5.895E 00 0.0\\ 2.558E 00 0.0\\ 2.558E 00 0.0\\ 3.405E-01 0.0\\ 1.089E-03 0.0\\ 1.943E-12 0.0\\ \end{array}$	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	$\begin{array}{c} 1.016E 05\\ 8.640E 04\\ 7.348E 04\\ 6.001E 04\\ 2.180E 04\\ 1.918E 03\\ 3.340E 03\\ 3.340E 03\\ 3.340E 03\\ 1.012E-02\\ 2.818E-13\\ 7.207E-31\\ 1.843E-48\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.$	$\begin{array}{c} 3.449E & 0.3\\ 3.360E & 0.3\\ 3.273E & 0.3\\ 3.273E & 0.3\\ 3.064E & 0.3\\ 3.064E & 0.3\\ 1.811E & 0.3\\ 2.687E & 0.3\\ 9.380FE & 0.2\\ 4.867E & 0.2\\ 4.867E & 0.0\\ 6.952E-0.3\\ 9.480E-26\\ 0.6\\ 0.6\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.$	$\begin{array}{c} 3.949E & 00 & 8.396E & 0\\ 3.949E & 00 & 7.963E & 0\\ 3.949E & 00 & 7.963E & 0\\ 3.949E & 00 & 7.963E & 0\\ 3.949E & 00 & 5.0599E & 0\\ 3.949E & 00 & 2.274E & 0\\ 3.949E & 00 & 2.274E & 0\\ 3.945E & 00 & 5.996E & 0\\ 3.942E & 00 & 4.169E & 0\\ 3.942E & 00 & 0.01 & 0\\ 3.942E & 00 & 0& 0\\ 3.942E & 0& 0& 0\\ 3.942E & 0& 0& 0& 0\\ 3.$	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	$\begin{array}{cccccccccccccccccccccccccccccccccccc$		

Radioactivity (Ci/MTIHM)

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inne												
(years)	CS137	PU241	TH229	AC225	RA225	FR221	AT217	BI 21 3	PB 20 9	U233	P0213	NP237
5.380E-01 7.380E-01 9.380E-01 1.238E 00 1.438E 00 2.438E 00 2.438E 00 1.044E 01 1.004E 01 1.004E 02 3.004E 02 3.004E 02 3.000E 03 3.000E 04 3.000E 04 3.000E 05 3.000E 05	1.351E 02 1.345E 02 1.339E 02 1.323E 02 1.2293E 02 1.2293E 02 1.206E 02 1.205E 02 1.205E 01 4.265E 01 1.3343E 01 4.265E 01 1.3343E 01 4.265E 01 1.3343E 01 4.265E 01 1.322E-01 1.3066E-28 0.0 0.0 0.0 0.0 0.0	2.130E 02 2.080E 02 2.080E 02 2.035E 02 2.035E 02 2.011E 02 1.916E 02 1.304E 02 8.057F 01 1.713E 00 1.543E-01 1.566E-04 2.566E-06 2.179E-06 1.231E-06 2.410E-07 2.440E-07 2.97E-10 6.589E-17 0.0	1.010E-09 1.010E-09 1.011E-09 1.011E-09 1.011E-09 1.012E-09 1.012E-09 1.023E-09 1.023E-09 1.452E-09 2.848E-08 1.770E-07 2.621E-06 3.103E-05 1.833E-04 1.581E-04 1.581E-04 1.581E-04	1.011E-09 1.011E-09 1.011E-09 1.011E-09 1.011E-09 1.012E-09 1.012E-09 1.023E-09 1.023E-09 1.3452E-09 2.848E-08 1.7452E-09 2.848E-08 1.770E-07 2.621E-06 3.103E-05 1.833E-04 1.581E-03	1.01 DE-09 1.01 DE-09 1.01 IE-09 1.01 IE-09 1.01 IE-09 1.01 2E-09 1.01 2E-09 1.01 2E-09 1.02 3E-09 1.02 3E-09 1.45 2E-09 2.14 7E-09 7.88 3E-09 2.84 8E-08 1.770E-07 2.62 IE-06 3.10 3E-05 1.83 3E-04 1.58 IE-03 1.74 9E-03 1.74 9E	rK221 1.011E-09 1.011E-09 1.011E-09 1.011E-09 1.012E-09 1.012E-09 1.012E-09 1.023E-09 1.0452E-09 2.147E-09 7.883E-09 2.848E-08 1.770E-07 2.621E-06 3.103E-05 1.833E-04 1.581E-04 1.5	A 1 61 7 1 01 1E-09 1 01 0E-09 1 01 1E-09 1 01 1E-09 1 01 1E-09 1 01 2E-09 1 01 2E-09 1 0128-09 1 0428-09 1 0452E-09 2 84 8E-08 1 0452E-09 2 84 8E-08 2 84 8E-08	1.213 1.011E-09 1.011E-09 1.011E-09 1.011E-09 1.012E-09 1.012E-09 1.012E-09 1.012E-09 1.023E-09 1.0452E-09 2.848E-08 1.746E-07 7.883E-09 2.848E-08 1.776E-07 3.103E-05 1.833E-04 1.581E-04 1.581E-04 1.581E-04	1.011E-09 1.011E-09 1.011E-09 1.011E-09 1.011E-09 1.012E-09 1.012E-09 1.012E-09 1.012E-09 1.012E-09 1.046E-09 1.0452E-09 2.044E-09 1.0452E-09 2.044E-08 1.770E-07 2.021E-06 3.032E-05 1.833E-04 1.5801E-04 1.5801E-04 1.5801E-04	0233 1.316E-08 1.322E-08 1.329E-08 1.3384E-08 1.3384E-08 1.384E-08 1.494E-08 1.494E-08 1.494E-08 1.494E-08 1.992E-07 6.605E-07 5.057E-06 2.362E-05 2.728E-05 2.728E-05 1.322E-05 2.728E-05 1.565E-03 1.745E-03	PU213 9.887E-10 9.887E-10 9.887E-10 9.893E-10 9.993E-10 9.993E-10 1.018E-09 1.018E-09 1.018E-09 2.1018E-09 2.787E-08 1.711E-07 2.785E-08 1.731E-07 2.565E-05 1.7731E-04 1.547E-03	NH237 7.6531E-05 7.6562E-05 7.6562E-05 7.715E-05 7.7488E-05 8.365E-05 9.322E-05 9.322E-05 1.168E-04 2.047E-04 3.604E-04 2.047E-04 3.604E-03 1.821E-03 2.255E-03 2.252E-03 2.2063E-03 2.2063E-03 2.2063E-03

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Appendix C.2. Thermal Power of FFTF Spent Fuel, High-Level Waste, and Structural Material Waste

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Fig. C.4. Thermal power of FFTF spent fuel as a function of decay time.

Thermal power (W/MTIHM)												
Time — (years)	TOTAL	PU239	AN241	PU240	RH106	PR 144	C S137	PU238	CM242	NB 95	SR 90	P0213
1.000E-01 3.000E-01 5.000E-01 1.000E 00 2.000E 00 1.000E 00 1.000E 01 1.000E 01 1.000E 01 1.000E 02 1.5000E 02 3.000E 02 1.5000E 03 3.000E 03 1.000E 04 1.000E 04 3.000E 05	1.611E 05 8.250E 04 5.410E 04 2.699E 04 1.210E 04 3.555E 03 2.264E 03 1.796E 03 1.492E 03 1.30E 03 1.102E 03 1.30E 03 1.102E 02 5.271E 02 5.271E 02 3.682E 02 2.223E 02 1.682E 02 2.223E 00	3.729E 02 3.729E 02 3.726E 02 3.717E 02 3.717E 02 3.696E 02 3.696E 02 3.696E 02 2.093E 02 2.093E 02	$\begin{array}{c} 2.474E & 02\\ 2.518E & 02\\ 2.562E & 02\\ 2.617E & 02\\ 2.671E & 02\\ 2.879E & 02\\ 3.446E & 02\\ 4.214E & 02\\ 5.259E & 02\\ 6.358E & 02\\ 6.358E & 02\\ 6.358E & 02\\ 5.792E & 02\\ 4.557E & 02\\ 3.307E & 02\\ 1.463E & 02\\ 6.000E & 02\\ 1.615E-04\\ 1.596E-08\\ 5.328E-08\\ 4.626E-15\\ \end{array}$	2.452E 02 2.452E 02 2.452E 02 2.452E 02 2.452E 02 2.452E 02 2.451E 02 2.451E 02 2.450E 02 2.4457E 02 2.4457E 02 2.4457E 02 2.4456E 02 2.414E 02 2.376E 02 2.376E 02 2.376E 02 1.784E 01 1.019E 01 6.090E-03 9.361E-10	$\begin{array}{c} 1.971E & 04\\ 1.717E & 04\\ 1.497E & 04\\ 1.261E & 04\\ 1.061E & 04\\ 5.337E & 02\\ 2.178E & 01\\ 2.476E-11\\ 2.895E-26\\ 3.385E-41\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.$	1.553E 04 1.300E 04 1.088E 04 8.705E 03 6.967E 03 2.859E 03 1.976E 02 2.301E 00 3.120E 00 3.120E 04 7.765E - 16 3.548E - 35 1.623E - 54 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.	$\begin{array}{c} \bullet.742E & 02\\ \bullet.711E & 02\\ \bullet.6602E & 02\\ \bullet.6402E & 02\\ \bullet.6402E & 02\\ \bullet.602E & 02\\ \bullet.5063E & 02\\ \bullet.5063E & 02\\ \bullet.5063E & 02\\ \bullet.2128E & 02\\ \bullet.2128E & 02\\ \bullet.2128E & 02\\ \bullet.2128E & 02\\ \bullet.243E - 08\\ \bullet.324E - 08\\ \bullet.324E - 08\\ \bullet.324E - 08\\ \bullet.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ \end{array}$	2.386E 02 2.539E 02 2.653E 02 2.747E 02 2.810E 02 2.8059E 02 2.869E 02 2.764E 02 2.564E 02 2.564E 02 2.564E 02 2.564E 02 2.564E 02 3.801E 01 3.346E 01 3.346E 01 3.546E 01 3.5997E-05 5.5482E-19 0.0 0.0	$\begin{array}{c} 1.283 \pm 04\\ 9.412 \pm 03\\ 6.937 \pm 03\\ 3.190 \pm 03\\ 3.190 \pm 03\\ 6.885 \pm 02\\ 2.232 \pm 01\\ 1.553 \pm 01\\ 1.483 \pm 01\\ 1.485 \pm 01\\ 1.865 \pm 01\\ 8.199 \pm 00\\ 1.665 \pm 01\\ 8.195 \pm 01\\ 1.866 \pm 05\\ 2.561 \pm -19\\ 0.0\\ 9.0\\ 0.3\\ 0.3\\ 0.3\\ 0.3\\ 0.3\\ 0.3\\ 0.3\\ 0$	2.322E 04 1.405E 04 2.867E 03 1.097E 03 1.207E 03 1.501E-04 3.837E-13 2.507E-30 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	3.525E 02 3.508E 02 3.470E 02 3.450E 02 3.450E 02 3.137E 02 2.784E 02 2.195E 02 3.269E 01 2.799E-01 2.397E-03 1.625E-08 3.441E-29 0.0 0.0 0.0	$1.112E-07\\1.002E-07\\9.973E-08\\9.973E-08\\9.974E-08\\9.974E-08\\1.003E-07\\1.010E-07\\1.010E-07\\1.026E-07\\1.026E-07\\1.109E-07\\2.112E-07\\2.805E-06\\1.745E-05\\2.587E-06\\1.745E-05\\3.063E-03\\1.810E-02\\7.096E-02\\1.561E-01\\1.561E-01\\1.561E-01\\1.561E-01\\1.561E-01\\1.561E-01\\1.561E-01\\1.561E-01\\1.561E-01\\1.561E-01\\1.561E-0\\1.561E-0\\1.561E-0\\1.551E-$
1.000E 06	1.249E 00	1.177E-10	0.0	9.268E-10	0.0	0.0	0.0	0.0	0.0	0.0	0.0	10/202-01

## Table C.4. Thermal power of FFTF spent fuel as a function of decay time

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Thermal power (W/MTIHM)

Time (years)	ZR 95	AT217	FR221	LA140	AC225	TH229	NP237	RU103	U233	CO 58	Y 91	U236
1.000E-01	1.880E 04	9.584E-08	8.669E-08	1.670E 04 3.187E 02	7.846E-08	6.160E-08	4.614E-03 4.632E-03	1.369E 04 3.771E 03	7.542E-07 7.581E-07	9.286E 03 4.542E 03	7.175E 03 3.021E 03	4.388E-04 4.401E-04
5.000E-01 8.000E-01	3.861E 03 1.436E 03	8.596E-08 8.596E-08	7.774E-08 7.774E-08	6.078E 00 4.310E-02	7.036E-08 7.036E-08	6.163E-08 6.163E-08	4.648E-03 4.666E-03	1.039E 03 2.075E 02	7.617E-07 7.669E-07	2.220E 03 9.078E 02	1.271E 03 4.307E 02	4.41 3E -04 4.42 8E -04
2.000E 00 5.000E 00	5.340E 02 1.021E 01 7.138E-05	8.599E-08 8.611E-08 8.641E-08	7.786E-08 7.816E-08	7.723E-13	7.048E-08 7.075E-08	6.172E-08 6.196E-08	4.768E-03 5.051E-03	6.690E-02 2.682E-10	7.934E-07 8.566E-07	1.038E 01 2.267E-04	1.929E 00 4.440E-06	4.509E-04 4.699E-04
1.000E 01 2.000E 01	1.825E-13 1.192E-30	8.702E-08 8.843E-08	7.870E-08 8.000E-08	0.0	7.123E-08 7.241E-08	6.238E-08 6.340E-08	5.626E-03 7.051E-03	2.714E-24 0.0	9.675E-07 1.229E-06 2.431E-06	3.873E-12 1.129E-27	1.783E-15 2.872E-34	5.012E-04 5.644E+04 7.530E-04
1.000E 02 1.500E 02	0.0	1.232E-07 1.820E-07	1.114E-07 1.646E-07	0.0	1.008E-07 1.490E-07	8-828E-08 1-305E-07	2.188E-02 3.084E-02	0.0	5.997E-06 1.149E-05	0.0	0.0	1.067E-03 1.378E-03
3.000E 02 5.000E 02	0.0	6.684E-07 2.418E-06	6.045E-07 2.187E-06	0.0	5.473E-07 1.979E-06 1.231E-05	4.792E-07 1.733E-06	5.388E-02 7.711E-02 1.110E-01	0.0	3.819E-05 9.310E-05 2.929E-04	0.0	0.0	2.303E-03 3.515E-03 6.431E-03
3.000E 03 1.000E 04	0.0	2.230E-04 2.640E-03	2.017E-04 2.388E-03	0.0	1.825E-04 2.161E-03	1.598E-04 1.893E-03	1.373E-01 1.381E-01	0.0	1.369E-03 5.292E-03	0.0	0.0	1.667E-02 3.937E-02
3.000E 04 1.000E 05 3.000E 05	0.0	1.560E-02 6.114E-02 1.345E-01	1.411E-02 5.530E-02 1.217E-01	0.0	1.277E-02 5.006E-02 1.101E-01	1.118E-02 4.382E-02 9.644E-02	1.372E-01 1.342E-01 1.257E-01	0.0	4.566E-02 9.372E-02	0.0	0.0	5.985E-02 5.949E-02
1.000E 06	0.0	1.488E-01	1.346E-01	0.0	1.218E-01	1.067E-01	1.002E-01	0.0	1.011E-01	0.0	0.0	5.828E-02

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Fig. C.5. Thermal power of FFTF high-level waste as a function of decay time.

Thermal power (W/MTIHM)												
Time (years)	TOTAL	AN241	PU239	PH106	CS137	PR144	SR 90	CM242	NP 237	P0213	PU240	AT217
	$\begin{array}{c} 4.723 \pm 0\\ 3.474 \pm 0\\ 2.714 \pm 0\\ 2.104 \pm 0\\ 1.688 \pm 0\\ 8.144 \pm 0\\ 8.144 \pm 0\\ 2.054 \pm 0\\ 1.189 \pm 0\\ 9.533 \pm 0\\ 6.039 \pm 0\\ 3.563 \pm 0\\ 2.640 \pm 0\\ 1.772 \pm 0\\ 1.2640 \pm 0\\ 1$	4       2.555E       0         4       2.555E       0         4       2.553E       0         4       2.553E       0         4       2.553E       0         3       2.553E       0         2       2.537E       0         2       2.489E       0         2       2.537E       0         2       1.57E       0         2       1.57E       0         0       1.539E       0         0       1.600E=0       0         1       3.603E=1       1         1       3.603E=1       1 <td>2 1.869E 00 2 1.889E 00 2 1.899E 00 2 1.89</td> <td><math display="block">\begin{array}{c} 1.462E &amp; 04\\ 1.274E &amp; 04\\ 1.110E &amp; 04\\ 9.350E &amp; 03\\ 7.973E &amp; 03\\ 5.030E &amp; 02\\ 1.665E - 02\\ 1.850E - 11\\ 1.685E - 02\\ 1.850E - 11\\ 2.164E - 26\\ 2.530E - 41\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.</math></td> <td><math display="block">\begin{array}{c} 6 &amp; 69  0E &amp; 02 \\ 6 &amp; 65  9E &amp; 02 \\ 6 &amp; 65  9E &amp; 02 \\ 6 &amp; 55  3E &amp; 02 \\ 6 &amp; 55  3E &amp; 02 \\ 5 &amp; 32  2E \\ 6 &amp; 4  03  E \\ 0 &amp; 2 \\ 5 &amp; 32  2E \\ 0 &amp; 2 \\ 1 &amp; 22  4E \\ 0 &amp; 2 \\ 2 &amp; 11  2E \\ 0 &amp; 2 \\ 1 &amp; 22  4E \\ 0 &amp; 2 \\ 2 &amp; 11  2E \\ 0 &amp; 2 \\ 1 &amp; 2 \\ 0 &amp; 9  6E \\ 0 &amp; 1 \\ 2 &amp; 0 &amp; 9  6E \\ 0 &amp; 1 \\ 2 &amp; 0 &amp; 9  6E \\ 0 &amp; 1 \\ 0 &amp; 1 &amp; 2E \\ 0 &amp; 0 \\ \end{array}</math></td> <td><math display="block">\begin{array}{c} 1.054E &amp; 04\\ 8.820E &amp; 03\\ 7.381E &amp; 03\\ 5.908E &amp; 03\\ 1.941E &amp; 03\\ 1.341E &amp; 02\\ 1.562E &amp; 00\\ 2.117E-04\\ 5.269E-16\\ 2.409E-35\\ 1.101E-54\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.</math></td> <td>3.497E 02 3.480E 02 3.463E 02 3.463E 02 3.443E 02 3.443E 02 3.112E 02 2.112E 02 2.178E 02 3.243E 01 9.866E-001 2.380E-03 1.615E-08 3.420E-03 3.420E-03 0.0 0.0 0.0</td> <td><math display="block">\begin{array}{c} 6.528E &amp; 03\\ 4.791E &amp; 03\\ 3.518E &amp; 03\\ 3.518E &amp; 03\\ 3.578E &amp; 02\\ 1.918E &amp; 01\\ 1.554E &amp; 01\\ 1.554E &amp; 01\\ 1.294E &amp; 01\\ 1.294E &amp; 01\\ 1.294E &amp; 01\\ 1.030E &amp; 01\\ 8.202E &amp; 00\\ 4.139E &amp; 00\\ 1.663E &amp; 00\\ 1.663E &amp; 00\\ 1.8652E - 19\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ \end{array}</math></td> <td><math display="block">\begin{array}{c} 4 &amp; 66 &amp; 2E - 03 \\ 4 &amp; 677E - 03 \\ 4 &amp; 697E - 03 \\ 4 &amp; 697E - 03 \\ 4 &amp; 712E - 03 \\ 4 &amp; 712E - 03 \\ 5 &amp; 034E - 03 \\ 5 &amp; 034E - 03 \\ 5 &amp; 411E - 02 \\ 1 &amp; 489E - 02 \\ 2 &amp; 314E - 02 \\ 1 &amp; 489E - 02 \\ 2 &amp; 107E - 02 \\ 3 &amp; 107E - 02 \\ 5 &amp; 214E - 02 \\ 5 &amp; 208E - 02 \\ 5 &amp; 298E - 02 \\ 5 &amp; 298E - 02 \\ 4 &amp; 771E - 02 \\ 3 &amp; 8 \\ 3 &amp; 3E - 02 \\ \end{array}</math></td> <td><math display="block">\begin{array}{c} 1 &amp; 0 &amp; 0 &amp; 0 &amp; 0 &amp; - &amp; 0 &amp; 7 \\ 9 &amp; 9 &amp; 9 &amp; 9 &amp; 7 &amp; - &amp; 0 &amp; 8 \\ 9 &amp; 9 &amp; 9 &amp; 9 &amp; 6 &amp; 0 &amp; - &amp; 0 &amp; 8 \\ 9 &amp; 9 &amp; 9 &amp; 6 &amp; 0 &amp; 0 &amp; 0 &amp; - &amp; 0 &amp; 8 \\ 9 &amp; 9 &amp; 9 &amp; 0 \\ 1 &amp; 0 &amp; 0 &amp; 5 &amp; 0 &amp; 0 &amp; 0 &amp; 0 &amp; 0 \\ 1 &amp; 0 &amp; 0 &amp; 5 &amp; 0 &amp; 0 &amp; 0 &amp; 0 &amp; 0 \\ 1 &amp; 0 &amp; 0 &amp; 5 &amp; 0 &amp; 0 &amp; 0 &amp; 0 &amp; 0 \\ 1 &amp; 0 &amp;</math></td> <td><math display="block">\begin{array}{c} 1 \cdot 229E &amp; 0 \\ 1 \cdot 229E &amp; 0 \\ 0 \\ 1 \cdot 230E &amp; 0 \\ 0 \\ 1 \cdot 230E &amp; 0 \\ 0 \\ 1 \cdot 232E &amp; 0 \\ 0 \\ 0 \\ 1 \cdot 232E &amp; 0 \\ 0 \\ 0 \\ 0 \\ 0 \\ 0 \\ 0 \\ 0 \\ 0 \\ 0</math></td> <td>8.620E-08 8.616E-08 8.616E-08 8.616E-08 8.616E-08 8.615E-08 8.62E-08 8.62E-08 8.62E-08 8.62E-08 8.62E-08 8.62E-08 8.62E-08 1.059E-07 1.385E-07 1.385E-07 1.385E-07 1.167E-06 6.419E-06 6.759E-05 1.010E-03 2.322E-02 5.106E-02 5.647E-02</td>	2 1.869E 00 2 1.889E 00 2 1.899E 00 2 1.89	$\begin{array}{c} 1.462E & 04\\ 1.274E & 04\\ 1.110E & 04\\ 9.350E & 03\\ 7.973E & 03\\ 5.030E & 02\\ 1.665E - 02\\ 1.850E - 11\\ 1.685E - 02\\ 1.850E - 11\\ 2.164E - 26\\ 2.530E - 41\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.$	$\begin{array}{c} 6 & 69  0E & 02 \\ 6 & 65  9E & 02 \\ 6 & 65  9E & 02 \\ 6 & 55  3E & 02 \\ 6 & 55  3E & 02 \\ 5 & 32  2E \\ 6 & 4  03  E \\ 0 & 2 \\ 5 & 32  2E \\ 0 & 2 \\ 1 & 22  4E \\ 0 & 2 \\ 2 & 11  2E \\ 0 & 2 \\ 1 & 22  4E \\ 0 & 2 \\ 2 & 11  2E \\ 0 & 2 \\ 1 & 2 \\ 0 & 9  6E \\ 0 & 1 \\ 2 & 0 & 9  6E \\ 0 & 1 \\ 2 & 0 & 9  6E \\ 0 & 1 \\ 0 & 1 & 2E \\ 0 & 0 \\ 0 & 0 \\ 0 & 0 \\ 0 & 0 \\ 0 & 0 \\ 0 & 0 \\ \end{array}$	$\begin{array}{c} 1.054E & 04\\ 8.820E & 03\\ 7.381E & 03\\ 5.908E & 03\\ 1.941E & 03\\ 1.341E & 02\\ 1.562E & 00\\ 2.117E-04\\ 5.269E-16\\ 2.409E-35\\ 1.101E-54\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.$	3.497E 02 3.480E 02 3.463E 02 3.463E 02 3.443E 02 3.443E 02 3.112E 02 2.112E 02 2.178E 02 3.243E 01 9.866E-001 2.380E-03 1.615E-08 3.420E-03 3.420E-03 0.0 0.0 0.0	$\begin{array}{c} 6.528E & 03\\ 4.791E & 03\\ 3.518E & 03\\ 3.518E & 03\\ 3.578E & 02\\ 1.918E & 01\\ 1.554E & 01\\ 1.554E & 01\\ 1.294E & 01\\ 1.294E & 01\\ 1.294E & 01\\ 1.030E & 01\\ 8.202E & 00\\ 4.139E & 00\\ 1.663E & 00\\ 1.663E & 00\\ 1.8652E - 19\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ \end{array}$	$\begin{array}{c} 4 & 66 & 2E - 03 \\ 4 & 677E - 03 \\ 4 & 697E - 03 \\ 4 & 697E - 03 \\ 4 & 712E - 03 \\ 4 & 712E - 03 \\ 5 & 034E - 03 \\ 5 & 034E - 03 \\ 5 & 411E - 02 \\ 1 & 489E - 02 \\ 2 & 314E - 02 \\ 1 & 489E - 02 \\ 2 & 107E - 02 \\ 3 & 107E - 02 \\ 5 & 214E - 02 \\ 5 & 208E - 02 \\ 5 & 298E - 02 \\ 5 & 298E - 02 \\ 4 & 771E - 02 \\ 3 & 8 \\ 3 & 3E - 02 \\ \end{array}$	$\begin{array}{c} 1 & 0 & 0 & 0 & 0 & - & 0 & 7 \\ 9 & 9 & 9 & 9 & 7 & - & 0 & 8 \\ 9 & 9 & 9 & 9 & 6 & 0 & - & 0 & 8 \\ 9 & 9 & 9 & 6 & 0 & 0 & 0 & - & 0 & 8 \\ 9 & 9 & 9 & 0 & 0 & 0 & 0 & 0 & 0 & 0 \\ 1 & 0 & 0 & 5 & 0 & 0 & 0 & 0 & 0 \\ 1 & 0 & 0 & 5 & 0 & 0 & 0 & 0 & 0 \\ 1 & 0 & 0 & 5 & 0 & 0 & 0 & 0 & 0 \\ 1 & 0 & 0 & 0 & 0 & 0 & 0 & 0 & 0 & 0 \\ 1 & 0 & 0 & 0 & 0 & 0 & 0 & 0 & 0 & 0 \\ 1 & 0 & 0 & 0 & 0 & 0 & 0 & 0 & 0 & 0 \\ 1 & 0 & 0 & 0 & 0 & 0 & 0 & 0 & 0 & 0 \\ 1 & 0 & 0 & 0 & 0 & 0 & 0 & 0 & 0 & 0 \\ 1 & 0 & 0 & 0 & 0 & 0 & 0 & 0 & 0 & 0 &$	$\begin{array}{c} 1 \cdot 229E & 0 \\ 1 \cdot 229E & 0 \\ 0 \\ 1 \cdot 230E & 0 \\ 0 \\ 1 \cdot 230E & 0 \\ 0 \\ 1 \cdot 232E & 0 \\ 0 \\ 0 \\ 1 \cdot 232E & 0 \\ 0 \\ 0 \\ 0 \\ 0 \\ 0 \\ 0 \\ 0 \\ 0 \\ 0$	8.620E-08 8.616E-08 8.616E-08 8.616E-08 8.616E-08 8.615E-08 8.62E-08 8.62E-08 8.62E-08 8.62E-08 8.62E-08 8.62E-08 8.62E-08 1.059E-07 1.385E-07 1.385E-07 1.385E-07 1.167E-06 6.419E-06 6.759E-05 1.010E-03 2.322E-02 5.106E-02 5.647E-02

### Table C.5. Thermal power of FFTF high-level waste as a function of decay time

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Thermal power (W/MTIHM)

Time												
(years)	FR221	AC225	TH229	U233	NB 95	PU238	ZR 95	U234	AM243	P0214	CE144	SB126M
5.380E-01 7.380E-01 9.380E-01 1.238E 00 1.438E 00 5.438E 00 5.438E 01 2.044E 01 2.044E 01 2.044E 02 3.004E 02 3.000E 03 1.000E 04 1.000E 05 3.000E 05	$\begin{array}{c} 7.796 \pm -08\\ 7.793 \pm -08\\ 7.793 \pm -08\\ 7.792 \pm -08\\ 7.799 \pm -08\\ 7.834 \pm -07\\ 1.253 \pm -07\\ 1.253 \pm -07\\ 1.434 \pm -07\\ 1.455 \pm -06\\ 5.798 \pm -06\\ 5.798 \pm -06\\ 5.798 \pm -06\\ 5.139 \pm -04\\ 3.435 \pm -05\\ 5.139 \pm -04\\ 3.455 \pm -03\\ 2.100 \pm -02\\ 4.618 \pm -02\\ 5.107 \pm -02\\ \end{array}$	$\begin{array}{c} 7.056E-08\\ 7.053E-08\\ 7.053E-08\\ 7.053E-08\\ 7.053E-08\\ 7.053E-08\\ 7.053E-08\\ 7.059E-08\\ 7.059E-08\\ 7.059E-08\\ 7.059E-08\\ 7.059E-08\\ 7.059E-08\\ 7.059E-08\\ 7.059E-08\\ 7.059E-08\\ 7.058E-07\\ 5.247E-06\\ 7.170E-05\\ 8.272E-04\\ 4.855E-03\\ 1.80E-02\\ 4.180E-02\\ 4.622E-02\\ \end{array}$	$\begin{array}{c} 6.177E-08\\ 6.177E-08\\ 6.177E-08\\ 6.177E-08\\ 6.177E-08\\ 6.177E-08\\ 6.177E-08\\ 6.177E-08\\ 6.172E-08\\ 6.172E-08\\ 7.592E-08\\ 9.931E-08\\ 9.931E-08\\ 9.931E-08\\ 7.592E-08\\ 9.931E-08\\ 7.592E-08\\ 1.592E-08\\ 1.59$	5.744E-09 9.621E-09 1.351E-08 1.351E-08 1.352E-08 1.087E-07 1.362E-07 1.362E-07 1.362E-06 5.25E-06 1.091E-04 2.074E-05 1.191E-04 5.295E-04 2.074E-03 1.318E-02 3.432E-03 1.332E-02 3.443E-03 3.432E-03 1.332E-02 3.443E-02 3.839E-02	6.303E 03 3.007E 03 1.399E 03 5.282E 02 1.977E 02 3.935E 00 2.659E-05 6.796E-14 4.441E-31 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.	$\begin{array}{c} 6.327E & 00\\ 1.429E & 01\\ 2.429E & 01\\ 2.873E & 01\\ 3.547E & 01\\ 3.547E & 01\\ 3.547E & 01\\ 3.466E & 01\\ 3.465E & 01\\ 2.873E & 01\\ 2.873E & 01\\ 2.873E & 01\\ 2.873E & 01\\ 3.589E & 00\\ 3.646E - 01\\ 3.6996E - 05\\ 5.485E - 19\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.$	$3 \cdot 330E 03$ $1 \cdot 509E 03$ $6 \cdot 841E 02$ $2 \cdot 544E 02$ $9 \cdot 459E 00$ $1 \cdot 265E - 05$ $3 \cdot 233E - 14$ $2 \cdot 112E - 31$ $0 \cdot 0$ $0 \cdot 0$ 0	$\begin{array}{c} 6 & 180 \pm -05 \\ 6 & 6998 \pm -05 \\ 7 & 553 \pm -05 \\ 8 & 961 \pm -04 \\ 8 & 5961 \pm -04 \\ 1 & 859 \pm -04 \\ 8 & 799 \pm -03 \\ 3 & 997 \pm -03 \\ 1 & 558 \pm -02 \\ 1 & 915 \pm -02 \\ 1 & 9$	$\begin{array}{c} 2 \cdot 324E - 01\\ 2 \cdot 323E - 01\\ 2 \cdot 319E - 01\\ 2 \cdot 319E - 01\\ 2 \cdot 302E - 01\\ 2 \cdot 291E - 01\\$	$\begin{array}{c} 3 & 6 & 70 \\ \overline{} & 6 & 70 \\ \overline{} & 8 & 26 \\ \overline{} & 8 & 49 \\ \overline{} & 7 & \overline{} & 8 \\ \overline{} & 8 & 49 \\ \phantom$	9.511E 02 7.959E 02 6.661E 02 5.331E 02 1.210E 02 1.210E 01 1.409E-C1 1.409E-C1 1.409E-C1 2.174E-36 9.937E- 56 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	2.533E-02 2.533E-02 2.533E-02 2.533E-02 2.533E-02 2.533E-02 2.533E-02 2.533E-02 2.533E-02 2.533E-02 2.533E-02 2.5531E-02 2.5524E-02 2.554E-02 2.554E-02 2.554E-02 2.554E-02 2.554E-02 2.554E-02 2.554E-02 2.554E-02 2.554E-02 2.554E-02 2.554E-02 2.554E-02 2.554E-02 2.554E-02 2.554E-02 2.5554E-02 2.555E

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Fig. C.6. Thermal power of FFTF structural material waste as a function of decay time.

					Thermal	power (W/MT	IHM)					
Time (years)	TOTAL	PU239	MN 54	AM241	PU240	NI 59	CO 58	CD 60	P0213	NI 63	PU238	AT217
5.380E-01 7.380E-01 9.380E-00 1.238E 000 1.438E 000 2.438E 000 2.438E 000 5.438E 000 1.044E 01 1.004E 02 1.004E 02 3.004E 02 3.004E 02 1.000E 04 1.000E 04 1.000E 04 1.000E 05 1.000E 05 1.000E 06	$\begin{array}{c} 2.540 \pm 0.3\\ 1.462 \pm 0.3\\ 9.058 \pm 0.2\\ 3.671 \pm 0.2\\ 1.638 \pm 0.2\\ 4.229 \pm 0.1\\ 5.141 \pm 0.0\\ 1.107 \pm 0.0\\ 8.450 \pm -0.1\\ 5.905 \pm -0.1\\ 3.882 \pm -0.1\\ 2.847 \pm -0.1\\ 2.018 \pm -0.1\\ 2.947 \pm -0.1\\ 3.892 \pm -0.2\\ 1.892 \pm -0.2\\ 2.230 \pm -0.4\\ 6.308 \pm -0.4\end{array}$	$\begin{array}{c} 1 \cdot 870E-01\\ 1 \cdot 869E-01\\ 1 \cdot 869E-02\\ 1 \cdot 780E-02\\ 1 \cdot 780E-02\\ 1 \cdot 780E-02\\ 3 \cdot 357E-05\\ 5 \cdot 989E-14\\ \end{array}$	5.059E 02 4.302E 02 2.659E 02 2.988E 02 1.085E 02 9.551E 00 1.663E-01 5.040E-05 1.403E-15 3.588E-33 9.178E-51 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.	$\begin{array}{c} 1 & 289 \\ \hline 1 & 311 \\ \hline - 01 \\ 1 & 331 \\ \hline 1 & 333 \\ \hline - 01 \\ 1 & 359 \\ \hline - 01 \\ 1 & 359 \\ \hline - 01 \\ 1 & 359 \\ \hline - 01 \\ 2 & 142 \\ \hline - 01 \\ \hline $	$\begin{array}{c} 1 \cdot 230 E - 01 \\ 1 \cdot 2230 E - 01 \\ 1 \cdot 2230 E - 01 \\ 1 \cdot 2230 E - 01 \\ 1 \cdot 223 E - 01 \\ 1 \cdot 233 E - 0$	1.824E-02 1.824E-02 1.824E-02 1.824E-02 1.824E-02 1.824E-02 1.824E-02 1.824E-02 1.822E-02 1.822E-02 1.822E-02 1.822E-02 1.816E-02 1.816E-02 1.816E-02 1.816E-02 1.672E-02 1.662E-03 1.356E-03 3.134E-06	$\begin{array}{c} 1.943 \\ \hline 9.503 \\ \hline 9.503 \\ \hline 8.503 \\ \hline$	5.318E 01 5.180E 01 5.046E 01 4.882E 01 4.724E 01 4.142E 01 2.792E 01 1.446E 01 3.881E 03 7.504E-02 1.072E-04 1.551E-07 1.072E-04 1.534E-27 0.0 0.0 0.0 0.0 0.0 0.0 0.0	5.00 3E-11 5.00 3E-11 5.00 3E-11 5.00 5E-11 5.01 3E-11 5.01 3E-11 5.066E-11 5.066E-11 5.066E-11 5.066E-11 5.066E-11 5.06E-10 5.06E-10	$\begin{array}{c} 1.627E-01\\ 1.624E-01\\ 1.622E-01\\ 1.619E-01\\ 1.619E-01\\ 1.568E-01\\ 1.5568E-01\\ 1.510E-01\\ 1.510E-01\\ 1.510E-01\\ 1.510E-01\\ 2.528E-02\\ 3.763E-02\\ 3.763E-03\\ 3.701E-05\\ 2.486E-11\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0$	$1 \cdot 338E-01 \\ 1 \cdot 376E-01 \\ 1 \cdot 403E-01 \\ 1 \cdot 426E-01 \\ 1 \cdot 459E-01 \\ 1 \cdot 459E-01 \\ 1 \cdot 381E-01 \\ 1 \cdot 381E-01 \\ 1 \cdot 382E-01 \\ 1 \cdot 382E-01 \\ 1 \cdot 381E-02 \\ 4 \cdot 859E-02 \\ 4 \cdot 859E-02 \\ 4 \cdot 859E-02 \\ 4 \cdot 313E-03 \\ 2 \cdot 309E-04 \\ 3 \cdot 309E-04 \\ $	4.312E-11 4.312E-11 4.313E-11 4.313E-11 4.313E-11 4.320E-11 4.337E-11 4.337E-11 4.367E-11 4.4601E-11 6.195E-11 9.163E-11 9.163E-10 1.215E-09 7.551E-09 7.551E-09 7.551E-09 7.551E-09 7.551E-09 7.551E-09 7.551E-09 7.551E-09 7.551E-09 7.551E-09 7.551E-09 7.551E-09 7.551E-09 7.551E-09 7.551E-09 7.551E-09 7.551E-09 7.551E-09 7.552E-06 7.462E-05
					Thermal	power (W/MT	IHM)					
Time (years)	FR221	CS137	AC225	TH229	NP237	U233	FE 55	SR 90	U236	NB 94	P0214	PU242
5.380E-01 7.380E-01 9.380E-01 1.238E 00 2.438E 00 5.438E 00 5.438E 00 1.044E 01 2.044E 01 1.004E 02 3.004E 02 3.004E 02 3.004E 02 3.004E 02 3.004E 02 3.004E 02 3.004E 02 3.000E 04 1.000E 04 1.000E 04 1.000E 04 1.000E 04 1.000E 05 3.000E 05 3.0	3.900E-11 3.902E-11 3.902E-11 3.907E-11 3.907E-11 3.907E-11 4.015E-11 4.015E-11 5.603E-11 8.288E-11 8.288E-11 8.043E-10 1.099E-09 1.012E-07 1.198E-06 5.2774E-05 5.6.749E-05	$\begin{array}{c} 3.347E-01\\ 3.331E-01\\ 3.316E-01\\ 3.296E-01\\ 3.298E-01\\ 3.298E-01\\ 2.989E-01\\ 2.632E-01\\ 2.632E-02\\ 1.056E-02\\ 3.228E-02\\ 1.048E-02\\ 3.228E-02\\ 3.278E-02\\ 3.278E-02\\ 3.278E-02\\ 3.278E-02\\ 3.278E-02\\ 0.06E-02\\ 0.00\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ $	3.630E-11 3.529E-11 3.532E-11 3.531E-11 3.537E-11 3.537E-11 3.557E-11 3.574E-11 3.574E-11 3.574E-11 3.574E-11 3.9574E-10 9.950E-10 6.181E-09 9.157E-08 1.084E-08 6.403E-08 5.52E-055 5.52E-055 5.52E-055	$\begin{array}{c} 3.090 \mbox{E} - 11\\ 3.091 \mbox{E} - 11\\ 3.092 \mbox{E} - 11\\ 3.093 \mbox{E} - 11\\ 3.093 \mbox{E} - 11\\ 3.097 \mbox{E} - 11\\ 3.109 \mbox{E} - 11\\ 3.109 \mbox{E} - 11\\ 3.130 \mbox{E} - 11\\ 3.143 \mbox{E} - 11\\ 3.441 \mbox{E} - 12\\ 3.441 \mbox{E} - 12\\ 3.441 \mbox{E} - 12\\ 3.441 \mbox{E} - 10\\ 5.69 \mbox{E} - 10\\ 5.41 \mbox{E} - 09\\ 5.40 \mbox{E} - 06\\ 3.49 \mbox{E} - 05\\ 5.350 \mbox{E} - 05\\ 5.350 \mbox{E} - 05\\ \end{array}$	$\begin{array}{c} 2.332 \pm -06\\ 2.340 \pm -06\\ 2.348 \pm -06\\ 2.348 \pm -06\\ 2.358 \pm -06\\ 2.557 \pm -06\\ 2.557 \pm -06\\ 2.849 \pm -06\\ 3.557 \pm -06\\ 1.102 \pm -05\\ 1.551 \pm -05\\ 3.869 \pm -05\\ 5.5687 \pm -05\\ 6.887 \pm -05\\ 6.887 \pm -05\\ 6.887 \pm -05\\ 6.305 \pm -05\\ 6.305 \pm -05\\ 5.026 \pm -0$	$\begin{array}{c} 3.825E-10\\ 3.844E-10\\ 3.864E-10\\ 3.864E-10\\ 3.913E-10\\ 4.022E-10\\ 4.022E-10\\ 4.022E-10\\ 4.022E-10\\ 6.231E-10\\ 1.231E-10\\ 3.027E-09\\ 5.790E-09\\ 5.790E-09\\ 5.790E-08\\ 4.677E-09\\ 5.654E-06\\ 5.073E-05\\ 5.078E-05\\ 5.07$	$\begin{array}{c} 1.155 \\ 1.095 \\ 0.095 \\ 0.083 \\ 0.083 \\ 0.083 \\ 0.083 \\ 0.083 \\ 0.083 \\ 0.083 \\ 0.083 \\ 0.003 \\ 0.033 \\$	$\begin{array}{c} 1.750E-01\\ 1.741E-01\\ 1.732E-01\\ 1.732E-01\\ 1.556E-01\\ 1.556E-01\\ 1.556E-01\\ 1.382E-01\\ 1.382E-01\\ 1.382E-02\\ 1.623E-02\\ 1.623E-02\\ 1.623E-02\\ 1.623E-02\\ 1.623E-02\\ 1.625E-02\\ 1.710E-32\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.$	$2 \cdot 215E - 07$ $2 \cdot 221E - 07$ $2 \cdot 227E - 07$ $2 \cdot 235E - 07$ $2 \cdot 235E - 07$ $2 \cdot 235E - 07$ $2 \cdot 275E - 07$ $2 \cdot 528E - 07$ $3 \cdot 79E - 07$ $5 \cdot 362E - 07$ $5 \cdot 362E - 07$ $1 \cdot 156E - 36$ $1 \cdot 764E - 06$ $3 \cdot 226E - 06$ $3 \cdot 26E - 06$ $2 \cdot 883E - 05$ $2 \cdot 982E - 05$	2.471E-03 2.471E-03 2.471E-03 2.471E-03 2.471E-03 2.471E-03 2.470E-03 2.470E-03 2.470E-03 2.4470E-03 2.4470E-03 2.4459E-03 2.462E-03 2.56E-04 2.56E-04 2.56E-03 2.56E-04 2.56E	$\begin{array}{c} 2 & 0 \\ 3 \\ 1 & 0 \\ 9 \\ 3 \\ 4 \\ 4 \\ -1 \\ 2 \\ 2 \\ 0 \\ 8 \\ 4 \\ -1 \\ 2 \\ 2 \\ -1 \\ 2 \\ 2 \\ -1 \\ 2 \\ 2 \\ -1 \\ 2 \\ -1 \\ 2 \\ -1 \\ 2 \\ -1 \\ 2 \\ -1 \\ 2 \\ -1 \\ 2 \\ -1 \\ 2 \\ -1 \\ -1$	$5 \cdot 80 5 \pm -05$ $5 \cdot 80 5 \pm -05$ $5 \cdot 80 5 \pm 05$ $5 \cdot 81 5 \pm 05$ $5 \cdot 82 3 \pm 05$ $5 \cdot 84 5 \pm 05$ $5 \cdot 85 2 \pm 05$ $5 \pm 05 2 \pm 05$ 5

#### Table C.6. Thermal power of FFTF structural material waste as a function of decay time

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Appendix C.3. Toxicity of FFTF Spent Fuel, High-Level Waste, and Structural Material Waste

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Fig. C.7. Toxicity of FFTF spent fuel as a function of decay time.

Toxicity (m <sup>3</sup> water/MTIHM)												
Time (years)	TOTAL	SR 90	PU239 PR1	43 PU240	RA226	AM241	RN219 RU	1106 CE144	P8210	TH229		
1.000E-01 3.000E-01 5.000E-01 1.000E 00 2.000E 00 2.000E 00 1.000E 01 1.000E 01 2.000E 01 1.000E 01 2.000E 01 3.000E 02 3.000E 02	1.787E 14 5.105E 12 7.361E 11 4.955E 11 4.955E 11 1.864E 11 1.864E 11 1.572E 11 6.681E 10 2.6685E 10 2.6685E 10 1.424E 1.0 7.753E 09	1.779E 11 1.770E 11 1.751E 11 1.751E 11 1.750E 11 1.700E 11 1.503E 11 1.503E 11 1.405E 11 1.405E 11 5.423E 10 1.650E 10 5.017E 09 1.413E 06	2.420E 09 1. 2.420E 09 4. 2.420E 09 9. 2.420E 09 9. 2.420E 09 8. 2.420E 09 7. 2.419E 09 0. 2.419E 09 0. 2.416E 09 0. 2.416E 09 0. 2.413E 09 0. 2.413E 09 0. 2.413E 09 0. 2.413E 09 0. 2.413E 09 0.	65E 14 1.575E 23E 12 1.575E 10E 11 1.575E 46E 06 1.575E 1.574E 1.574E 1.572E 1.5574E 1.5574E 1.5572E 1.556E 1.556E 1.526E	09 2.419E 00 09 2.518E 00 09 2.620E 00 09 2.749E 00 09 2.882E 00 09 3.452E 00 09 3.452E 00 09 1.040E 01 09 1.040E 01 09 1.549E 02 09 7.687E 02 09 7.687E 02 09 2.886E 00 09 1.136E 04 09 3.801E 00	1.862E 09 1.895E 09 1.928E 09 2.010E 09 2.010E 09 2.167E 09 2.572E 09 2.572E 09 1.3.958E 09 2.4.786E 09 2.4.786E 09 2.4.696E 09 3.4.28E 09 3.4.28E 09	1.448E 01 2. 1.387E 01 1. 1.503E 01 1. 1.575E 01 1. 1.909E 01 5. 2.864E 01 7. 4.657E 01 2. 8.873E 01 2. 2.494E 02 2. 5.750E 02 3. 9.349E 02 3. 2.148E 03 0.	055E       11       2.113E       11         791E       11       1.768E       11         561E       11       1.479E       11         314E       11       1.184E       11         107E       11       9.479E       10         563E       10       3.688E       10         069E       09       2.6689E       09         271E       08       3.132E       07         350E       05       4.244E       03         530E-34       1.056E-08       018E-19       4.828E-28         530E-34       2.207E-747       0.00	1.661E 00 9 1.698E 00 9 1.690E 00 9 1.683E 00 9 1.661E 00 9 1.632E 00 9 1.736E 00 9 1.736E 00 9 1.731E 01 1 1.196E 02 3 3.846E 03 0 2.6646E 03	5.033E 00 5.033E 00 5.036E 00 5.039F 03 5.04 5E 00 5.04 5E 00 5.096E 00 5.181E 00 5.599E 00 7.217E 00 1.066E 01 3.916E 01		
1.000E 02 3.000E 03 1.000E 04 3.000E 04 1.000E 05 3.000E 05 1.000E 06	4.894E 09 3.419E 09 2.378E 09 1.135E 09 2.485E 08 1.318E 08 5.943E 07	0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	2.351E 09 0.0 2.220E 09 0.0 1.814E 09 0.0 1.020E 09 0.0 1.358E 08 0.0 4.280E 05 0.0 7.638E-04 0.0	1 • 4 4 7E 1 • 1 4 6E 5 • 4 5 5E 6 • 54 5E 3 • 91 2E 5 • 95 5E	09 1.747E 0 09 1.426E 0 08 8.497E 0 07 2.969E 0 04 6.696E 0 -03 7.199E 0	244062 09 5 1.16E 09 5 4.515E 07 5 1.216E 03 7 1.201E 02 7 4.012E-01 7 3.482E-08 7 0.0	1.067E 03 J. 1.067E 04 0. 5.497E 04 0. 4.151E 05 0. 2.479E 06 0. 9.940E 06 0. 1.401E 07 0.	0         0.0           0         0.0           0         0.0           0         0.0           0         0.0           0         0.0           0         0.0           0         0.0           0         0.0           0         0.0           0         0.0	9.8376 03 5.2386 04 4.277E 05 2.548E 06 8.903E 06 2.007E 07 2.159E 07 5.527E 06	1.41/E 02 8.810E 02 1.306E 04 1.547E 05 9.138E 05 3.581E 06 7.882E 06 8.717E 06		

Table C.7.	Toxicity of	FFTF	spent	fuel	as a	function	of	decay	time
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Toxicity (m<sup>3</sup> water/MTIHM)

Time (years)	RA225	C5137	PU238	NP237	P0210	I 1 2 9	SR 89	PU241	AC225	RA223	CM242	PA231
$\begin{array}{c} 1.000E-01\\ 3.000E-01\\ 5.000E-01\\ 1.000E 00\\ 2.000E 00\\ 5.000E 00\\ 5.000E 00\\ 1.000E 01\\ 1.000E 02\\ 1.500E 02\\ 1.500E 02\\ 3.000E 02\\ 1.500E 02\\ 3.000E 02\\ 1.000E 02\\ 1.000E 02\\ 1.000E 02\\ 1.000E 02\\ 1.000E 02\\ 0.00E 02\\ $	$\begin{array}{c} 4.238 \pm 0 \\ 4.033 \pm 0 \\ 4.033 \pm 0 \\ 4.030 \pm 0 \\ 4.051 \pm 0 \\ 4.078 \pm 0 \\ 4.078 \pm 0 \\ 4.078 \pm 0 \\ 5.771 \pm 0 \\ 8.530 \pm 0 \\ 5.771 \pm 0 \\ 8.530 \pm 0 \\ 1.133 \pm 0 \\ 1.133 \pm 0 \\ 1.238 \pm 0 \\$	$\begin{array}{c} 6.991E \\ 0.928E \\ 0.90 \\ 0.64244E \\ 0.90 \\ 0.5563E \\ 0.90 \\ 0.4416E \\ 0.90 \\ $	1.440E 09 1.533E 09 1.553E 09 1.659E 09 1.658E 09 1.752E 09 1.752E 09 1.752E 09 1.752E 09 1.523E 09 1.523E 08 2.019E	S. 033E 04 5.051E 04 5.090E 04 5.090E 04 5.111E 04 5.122E 04 5.2509E 04 5.202E 04 5.2387E 05 3.364E 05 5.3764E 05 5.876E 05 5.876E 05 1.410E 06 1.4507E 06 1.4507E 06 1.4507E 06	$1 \cdot 24 \cdot 2E - 01$ $1 \cdot 58 \cdot 9E - 01$ $2 \cdot 03 \cdot 7E - 01$ $2 \cdot 03 \cdot 7E - 01$ $2 \cdot 31 \cdot 9E - 01$ $2 \cdot 32 \cdot 9E - 01$ $2 \cdot 32 \cdot 9E - 01$ $2 \cdot 34 \cdot 9E - 01$ $2 \cdot 48 \cdot 2E - 01$ $3 \cdot 90 \cdot 4E - 01$ $3 \cdot 90 \cdot 4E - 01$ $3 \cdot 78 \cdot 6E - 01$ $3 \cdot 64 \cdot 2E - 03$ $5 \cdot 12 \cdot 72 \cdot 6E - 03$ $2 \cdot 86 \cdot 8E - 06$ $3 \cdot 08 \cdot 4E - 06$	1.016E 06 1.024E 06 1.027E	$\begin{array}{c} 4.608E 11\\ 1.691E 11\\ 5.202E 10\\ 1.771E 10\\ 5.057E 09\\ 3.364E 07\\ 9.879E 00\\ 1.282E-10\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0$	2.138E 09 2.118E 09 2.072E 09 2.072E 09 2.072E 09 1.951E 09 1.328E 09 1.328E 09 8.205E 08 1.974E 07 1.571E 06 1.176E 03 2.672E 01 2.578E	$\begin{array}{c} 4.491E-01\\ 4.030E-01\\ 4.030E-01\\ 4.030E-01\\ 4.030E-01\\ 4.030E-01\\ 4.336E-01\\ 4.336E-01\\ 4.479E-01\\ 5.771E-01\\ 8.530E-01\\ 3.133E\\ 01\\ 1.045E\\ 03\\ 1.238E\\ 03\\ 1.238E\\ 03\\ 1.238E\\ 03\\ 1.238E\\ 04\\ 2.866E\\ 04\\ 2.866E\\ 05\\ 05\\ 05\\ 05\\ 05\\ 05\\ 05\\ 05\\ 05\\ 05$	6.205E-01 5.946E-01 6.443E-01 6.750E-01 6.750E-01 1.227E 00 1.996E 00 1.996E 00 1.996E 00 1.996E 00 1.9069E 01 2.4664E 01 1.669E 01 2.4664E 01 1.736E 02 4.575E 02 4.575E 02 4.575E 03 1.779E 02 4.56E 03 1.779E 02 4.56E 03 1.926E 05 0.926E	1.741E 10 1.277E 10 9.377E 10 9.367E 09 4.328E 09 4.328E 09 3.027E 07 2.107E 07 2.107E 07 2.107E 07 1.397E 07 1.112E 07 5.255E 06 2.255E 06 2.552E 01 3.476E-13 0.0 0.0	3.379E 00 3.422E 000 3.521E 000 3.578E 000 3.795E 000 3.795E 000 5.542E 000 5.542E 000 5.542E 000 1.454E 01 3.895E 01 1.458E 02 1.832E 03 1.832E 04 3.554E 02 1.832E 04 3.513E 05
1.000E 06	6.973E 0	5 0.0	0.0	1.093E 06	7.897E 05	9.825E 05	0.0	0.0	6.973E 05	6.036E 05	0.0	4.696E 05

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Fig. C.8. Toxicity of FFTF high-level waste as a function of decay time.

	Toxicity (m <sup>3</sup> water/MTIHM)											
Time — (years)	TOTAL	SR 90	AM241	RA226	PU239	RU106	TH229	CE144	RA225	P8210	PU240	NP237
$5.380 \pm 01$ $7.380 \pm 01$ $9.380 \pm 01$ $1.238 \pm 00$ $1.438 \pm 00$ $2.438 \pm 00$ $1.438 \pm 00$ $2.438 \pm 01$ $1.044 \pm 01$ $1.044 \pm 01$ $1.004 \pm 02$ $3.004 \pm 02$ $3.004 \pm 02$ $3.004 \pm 03$ $1.000 \pm 03$ $3.000 \pm 03$	$\begin{array}{c} 6.447E & 11\\ 4.910E & 11\\ 4.272E & 11\\ 3.753E & 11\\ 3.753E & 11\\ 2.498E & 11\\ 1.743E & 11\\ 1.4743E & 11\\ 1.$	1.765E 11 1.776E 11 1.778E 11 1.738E 11 1.687E 11 1.570E 11 1.570E 11 1.395E 11 1.395E 11 1.397E 10 1.637E 10 4.977E 08 1.202E 08 1.202E 08 1.202E 08 1.202E 08 1.202E 08 0.120E-20 0.0 0.0 0.0 0.0 0.0	1.923E 09 1.922E 09 1.922E 09 1.921E 09 1.921E 09 1.921E 09 1.911E 09 1.911E 09 1.873E 09 1.653E 09 1.526E 09 1.526E 09 8.706E 08 3.905E 08 1.580E 07 8.260E 02 1.204E 02 3.993E-01 3.465E-08 0.0	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	$\begin{array}{c} 1 \cdot 21 \ 3 \mathbb{E} & 07 \\ 1 \cdot 21 \ 3 \mathbb{E} & 07 \\ 1 \cdot 21 \ 3 \mathbb{E} & 07 \\ 1 \cdot 21 \ 3 \mathbb{E} & 07 \\ 1 \cdot 21 \ 3 \mathbb{E} & 07 \\ 1 \cdot 21 \ 3 \mathbb{E} & 07 \\ 1 \cdot 21 \ 3 \mathbb{E} & 07 \\ 1 \cdot 21 \ 3 \mathbb{E} & 07 \\ 1 \cdot 21 \ 3 \mathbb{E} & 07 \\ 1 \cdot 21 \ 3 \mathbb{E} & 07 \\ 1 \cdot 21 \ 0 \mathbb{E} & 07 \\ 1 \cdot 21 \ 0 \mathbb{E} & 07 \\ 1 \cdot 21 \ 0 \mathbb{E} & 07 \\ 1 \cdot 20 \ 0 \mathbb{E}$	1.524E 11 1.328E 11 1.158E 11 9.749E 10 8.209E 10 8.245E 08 1.756E 05 1.929E-04 2.256E-19 2.638E-34 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.	$\begin{array}{c} 5.048E & 00\\ 5.048E & 00\\ 5.048E & 00\\ 5.048E & 00\\ 5.047E & 00\\ 5.047E & 00\\ 5.052E & 00\\ 5.052E & 00\\ 5.074E & 00\\ 6.204E & 00\\ 8.115E & 00\\ 8.115E & 00\\ 2.224E & 01\\ 6.835E & 02\\ 5.131E & 03\\ 5.131E & 03\\ 5.131E & 04\\ 5.1360E & 04\\ 3.475E & 05\\ 1.360E & 06\\ 3.308E & 06\\ \end{array}$	$\begin{array}{c} 1.434E 11\\ 1.200E 11\\ 1.004E 11\\ 8.037E 10\\ 6.433E 10\\ 2.640E 10\\ 1.825E 07\\ 2.125E 07\\ 2.880E 07\\ 3.277E-28\\ 1.498E-47\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.$	$\begin{array}{c} 4.039E\\ 0.038E\\ 0.038E\\ 0.038E\\ 0.038E\\ 0.038E\\ 0.04\\ 0.038E\\ 0.04\\ 0.038E\\ 0.04\\ 0.038E\\ 0.04\\ 0.038E\\ 0.04\\ 0.038E\\ 0.04\\ 0.038E\\ 0.0$	$\begin{array}{c} 1.699E & GO \\ 1.6693E & OO \\ 1.6693E & OO \\ 1.6682E & OO \\ 1.676E & OO \\ 1.675E & OO \\ 1.968E & OI \\ 1.968E$	$\begin{array}{c} 7.895 \\ \hline 0.895 \\ \hline 0.895 \\ \hline 0.895 \\ \hline 0.895 \\ \hline 0.898 \\ \hline 0.898 \\ \hline 0.898 \\ \hline 0.890 \\ \hline 0.830 \\ \hline$	5.085E 04 5.101E 04 5.139E 04 5.139E 04 5.242E 04 5.242E 04 5.242E 04 6.716E 04 6.716E 04 6.716E 04 6.716E 05 1.624E 05 1.624E 05 1.624E 05 5.502E 05 5.687E 05 5.687E 05 5.680E 05 5.52E 05 5.204E 05 5.20

Table C.8. Toxicity of FFTF high-level waste as a function of decay time

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Toxicity (m<sup>3</sup> water/MTIHM)

Time (years)	C\$137	SR 89	PR143	SN126	AM243	P0210	AC225	CM242	NB 95	PD107	ZR 95	Y 91
5.380E-01 7.380E-01 9.380E-01 9.380E-01 1.238E 00 2.438E 00 2.438E 00 2.438E 00 1.044E 01 5.044E 01 5.044E 01 1.004E 02 3.004E 02 3.004E 02 3.0004E 03 3.0004E 03 1.0005 04 3.0005 04	6.939E 09 6.907E 09 6.875E 09 6.875E 09 6.636E 09 6.796E 09 6.196E 09 5.520E 09 4.382E 09 2.191E 09 2.191E 08 6.791E 08 6.791E 08 6.684E 04 6.422E-01 5.476E-21 0.0 0.0	5.139E 10 1.885E 10 6.917E 09 1.975E 09 5.639E 08 3.748E 00 1.101E 00 1.429E-11 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	4.978E 10 1.191E 09 2.850E 07 2.6850E 07 2.522E 03 1.978E-05 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0	6.632E 05 6.632E 05 6.632E 05 6.632E 05 6.632E 05 6.632E 05 6.632E 05 6.632E 05 6.631E 05 6.631E 05 6.631E 05 6.631E 05 6.625E 05 6.625E 05 6.6495E 05 6.6495E 05 6.495E 05 6.49	1.807E 06 1.807E 06 1.807E 06 1.807E 06 1.807E 06 1.807E 06 1.807E 06 1.807E 06 1.804E 06 1.804E 06 1.799E 06 1.799E 06 1.757E 06 1.757E 06 1.645E 06 1.363E 06 7.065E 05 1.0080E 05 1.501E 02	1.880E-01 2.036E-01 2.142E-01 2.277E-01 2.335E-01 2.338E-01 2.338E-01 2.964E-01 7.733E-01 2.81E 00 7.378E 00 4.727E 01 1.861E 02 1.105E 03 1.000E 04 6.172E 04 2.169E 05 4.864E 05	$\begin{array}{c} 4.040 = -01\\ 4.038 = -01\\ 4.038 = -01\\ 4.038 = -01\\ 4.038 = -01\\ 4.038 = -01\\ 4.038 = -01\\ 4.038 = -01\\ 4.038 = -01\\ 4.042 = -01\\ 6.492 = -01\\ 6.492 = -01\\ 1.779 = 00\\ 5.468 = 00\\ 3.004 = 01\\ 4.105 = 02\\ 4.736 = 03\\ 2.780 = 03\\ 2.780 = 03\\ 1.088 = 05\\ \end{array}$	8.859E 09 6.502E 09 4.773E 09 3.246E 09 2.209E 07 2.103E 07 2.013E 07 1.756E 07 1.398E	1.314E 10 6.270E 09 2.916E 09 4.122E 08 8.203E 08 5.543E 01 1.417E-07 9.258E-25 0.0 0.9 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	1.002E 0 1.002E 0 1.0002E 0	5 1.096E 10 5 4.967E 09 5 8.251E 09 5 8.370E 08 5 5.951E 08 5 5.951E 08 5 5.951E 01 5 5.951E 01 5 5.951E 01 5 6.955 5 0.0 5 5 5 0.0 5 5 5 0.0 5 5 5 0.0 5 5 5 5 0.0 5 5 5 5 5 5 5 5 5 5 5 5 5 5 5 5 5 5 5	1.003E 13 4.221E 09 1.772E 08 2.042E 08 2.696E 00 2.491E-09 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0
3.000E 05 1.000E 06	0.0	0.0	0.0	8.291E 04 6.475E 02	1.965E-05 1.803E-05	5.056E 05 8.351E 04	2.393E 05 2.646E 05	0.0	0.0	9.002E 0	4 0 • 0	2.0



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Fig. C.9. Toxicity of FFTF structural material waste as a function of decay time.

### Table C.9. Toxicity of FFTF structural material waste as a function of decay time

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Toxicity (m <sup>3</sup> water/MTIHM)												
Time — (years)	TOTAL	PU239	SR 90	C0 58	MN 54	RA226	PU240	NI 63	CD 60	AM241	RN219	P821 0
5.380E-01 7.380E-01 9.380E-01 1.238E 000 2.438E 000 5.438E 000 1.044E 01 1.044E 01 1.044E 02 1.004E 02 1.004E 02 1.004E 02 1.004E 02 1.000E 03 1.000E 04 1.000E 04 1.000E 04 1.000E 04 1.000E 05 1.	5.096E 09 3.017E 09 1.953E 09 1.275E 09 9.328E 08 4.609E 08 1.8869E 08 1.232E 08 8.282E 07 1.295E 07 1.995E 07 1.167E 06 3.690E 06 1.632E 06 1.632E 06 1.632E 06 1.632E 06 1.330E 05 5.678E 04 2.933E 04	1.214E 06 1.214E 06 1.213E 06 1.208E 06 1.179E 06 1.179E 06 1.179E 06 2.178E 06 2.178E 07 2.3887E-07	8.827E 07 8.786E 07 8.786E 07 8.692E 07 8.641E 07 7.856E 07 7.856E 07 5.497E 07 2.692E 07 8.188E 06 2.4491E 06 0.2491E 06 0.2491E 06 0.2491E 06 0.0075E-03 8.630E-24 0.00 0.00	$\begin{array}{cccccccc} 3.607E & 09\\ 1.764E & 09\\ 8.624E & 08\\ 3.526E & 08\\ 1.442E & 08\\ 4.031E & 06\\ 8.807E & 01\\ 1.504E-06\\ 4.386E-22\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0$	$\begin{array}{c} 1.016E & 09\\ 8.640E & 08\\ 7.348E & 08\\ 6.001E & 08\\ 2.1B0E & 08\\ 1.918E & 07\\ 3.340E & 05\\ 1.012E & 09\\ 7.207E-27\\ 1.843E-09\\ 7.207E-27\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.$	$\begin{array}{c} 1.324E-03\\ 1.376E-03\\ 3.429E-03\\ 1.497E-03\\ 1.865FE-03\\ 2.948E-03\\ 3.965E-02\\ 3.895E-01\\ 1.395E-02\\ 3.895E-01\\ 1.046E\\ 0.0\\ 1.910E\\ 0.0\\ 1.910E\\ 0.0\\ 1.910E\\ 0.0\\ 3.58E\\ 0.0\\ 1.489E\\ 0.4\\ 3.58E\\ 0.4\\ 3.611E\\ 0.4\\ 0.245E\\ 0.3\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0$	$\begin{array}{c} 7.899E \ 05\\ 7.899E \ 05\\ 7.899E \ 05\\ 7.898E \ 05\\ 7.898E \ 05\\ 7.898E \ 05\\ 7.897E \ 05\\ 7.895E \ 05\\ 7.895E \ 05\\ 7.857E \ 05\\ 7.857E \ 05\\ 7.775E \ 05\\ 7.775E \ 05\\ 7.775E \ 05\\ 7.736E \ 05\\ 7.736E \ 05\\ 3.282E \ 04\\ 1.962E \ 04\\ 1.962E \ 04\\ 1.962E \ 05\\ 2.986E \ 06\\ 2.986E \ 06\\ \end{array}$	$\begin{array}{c} 1.365E & 07\\ 1.363E & 07\\ 1.363E & 07\\ 1.359E & 07\\ 1.359E & 07\\ 1.346E & 07\\ 1.346E & 07\\ 1.267E & 07$	$\begin{array}{c} 1.150E & 08\\ 1.120E & 08\\ 3.09EE & 08\\ 1.056E & 08\\ 1.056E & 08\\ 3.955E & 07\\ 6.035E & 07\\ 3.127E & 07\\ 3.127E & 07\\ 3.127E & 02\\ 3.317E & 02\\ 3.317E & 02\\ 3.316E-21\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.$	9.703E 05 9.867E 05 1.003E 06 1.023E 06 1.023E 06 1.120E 06 1.329E 06 2.402E 06 2.353E 06 2.353E 06 2.353E 06 1.718E 06 1.718E 06 1.718E 06 1.718E 06 5.592E 05 2.263E 04 6.094E-01 1.6025E-02 1.999E-04 1.736E-11 0.0	$\begin{array}{c} 7.522 \pm -033\\ 7.5522 \pm -033\\ 8.539 \pm -033\\ 8.539 \pm -023\\ 8.539 \pm -023\\ 1.5510 \pm -022\\ 4.520 \pm -022\\ 4.520 \pm -022\\ 4.520 \pm -012\\ 2.420 \pm -022\\ 4.520 \pm -012\\ 2.420 \pm -022\\ 4.520 \pm$	$\begin{array}{c} 8.501E-04\\ 8.474E-04\\ 8.48E-04\\ 8.48E-04\\ 8.387E-04\\ 8.287E-04\\ 8.287E-04\\ 8.287E-04\\ 8.287E-04\\ 8.287E-04\\ 1.407E-03\\ 8.880E-04\\ 1.880E-04\\ 1.880E-03\\ 0.012E-03\\ 0.012E$

Toxicity (m<sup>3</sup> water/MTIHM)

lime												
(years)	TH229	R4225	NB 94	NI 59	RU196	CE144	CS137	PU238	MO 93	NP237	P0210	FE 55
5.380E-01 7.380E-01 1.238E 00 1.438E 00 2.438E 00 1.044E 01 1.004E 02 1.004E 02 1.004E 02 1.004E 02 1.004E 02 1.004E 02 1.004E 02 1.004E 03 1.000E 03 1.000E 04 1.000E 04 1.000E 05 1.000E 05	$\begin{array}{c} 2.525 \pm -03\\ 2.526 \pm -03\\ 2.527 \pm -03\\ 2.527 \pm -03\\ 2.528 \pm -03\\ 2.528 \pm -03\\ 2.5341 \pm -03\\ 2.541 \pm -03\\ 2.5641 \pm -03\\ 2.5641 \pm -03\\ 3.629 \pm 03\\ 3.629 \pm 03\\ 3.953 \pm 02\\ 1.797 \pm 03\\ 3.952 \pm 03\\ 3.95$	$\begin{array}{c} 2.021 \pm -03\\ 2.021 \pm -03\\ 2.021 \pm -03\\ 2.022 \pm -03\\ 2.041 \pm -03$	$\begin{array}{c} 8.082E & 04\\ 8.081E & 04\\ 8.081E & 04\\ 8.069E & 04\\ 8.069E & 04\\ 8.055E & 04\\ 8.055E & 04\\ 8.055E & 04\\ 8.055E & 04\\ 7.946E & 04\\ 7.946E & 04\\ 7.296E & 04\\ 7.296E & 04\\ 7.296E & 04\\ 2.902E & 04\\ 2.947E & 00\\ 1.227E-10\\ \end{array}$	$\begin{array}{c} 1.432E & 04\\ 1.39E & 0$	$\begin{array}{c} 7.625 \pm 0.7\\ 6.645 \pm 0.7\\ 5.791 \pm 0.7\\ 4.877 \pm 0.7\\ 2.065 \pm 0.7\\ 2.6624 \pm 0.6\\ 8.428 \pm 0.4\\ 8.778 \pm 0.8\\ 1.127 \pm -2.2\\ 1.318 \pm -37\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.0\\ 0.$	$\begin{array}{c} 7 \cdot 173 \ \ \ \ \ \ \ \ \ \ \ \ \ \ \ \ \ \ \$	$\begin{array}{c} 3.471E 06\\ 3.455E 06\\ 3.439E 06\\ 3.420E 06\\ 3.420E 06\\ 3.322E 06\\ 3.322E 06\\ 3.322E 06\\ 3.322E 06\\ 2.762E 06\\ 2.762E 06\\ 3.452E 05\\ 1.0087E 05\\ 3.397E 05\\ 3.394E 05\\ 3.394E 05\\ 3.394E 05\\ 3.394E 05\\ 0.087E 05\\ 0.0$	$B \cdot 0.74E 05$ $B \cdot 3.02E 055$ $B \cdot 465E 055$ $B \cdot 605E 055$ $B \cdot 605E 055$ $B \cdot 605E 055$ $B \cdot 63E 055$ $B \cdot 3.33E 055$ $T \cdot 7.35E 055$ $2 \cdot 956E 055$ $2 \cdot 956E 055$ $2 \cdot 60.38E 055$ $1 \cdot 20.956E 055$ $2 \cdot 60.38E 051$ $1 \cdot 20.7E - 011$ $1 \cdot 656 - 155$ $0 \cdot 0$ $0 \cdot 0$ $0 \cdot 0$	5.431E 04 5.430E 04 5.430E 04 5.430E 04 5.425E 04 5.425E 04 5.425E 04 5.422E 04 5.422E 04 5.324E 04 5.3	$\begin{array}{c} 2.544E & 01\\ 2.552E & 01\\ 2.552E & 01\\ 2.572E & 01\\ 2.572E & 01\\ 2.583E & 01\\ 2.583E & 01\\ 2.788E & 01\\ 3.1078E & 01\\ 3.1078E & 01\\ 3.1078E & 02\\ 2.950E & 02\\ 4.219E & 02\\ 4.219E & 02\\ 4.219E & 02\\ 4.219E & 02\\ 4.555E & 02\\ 7.555E & 02\\ 7.555E & 02\\ 7.538E & 02\\ 5.482E & 02\\ 5.482E & 02\\ \end{array}$	$\begin{array}{c} 9.404 \pm -05\\ 1.019 \pm -04\\ 1.019 \pm -04\\ 1.114 \pm -04\\ 1.114 \pm -04\\ 1.162 \pm -04\\ 1.162 \pm -04\\ 1.258 \pm -03\\ 2.010 \pm -03$	$\begin{array}{c} 1.049 \\ \hline 0.49 \\ \hline 9.950 \\ \hline 9.433 \\ \hline 0.8256 \\ \hline 0.6324 \\ $

Appendix D. COMPARISON OF CHARACTERISTICS OF FUEL CYCLE MATERIALS FROM A SINGLE REACTOR

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Appendix D.1. Comparison of the Radioactivity of Fuel Cycle Materials from a Single Reactor







Fig. D.2. Radioactivity of LMFBR spent fuel, high-level waste, and structural material waste as a function of decay time.





Appendix D.2. Comparison of the Thermal Power of Fuel Cycle Materials from a Single Reactor



Fig. D.4. Thermal power of PWR spent fuel, high-level waste, and structural material waste as a function of decay time.



Fig. D.5. Thermal power of LMFBR spent fuel, high-level waste, and structural material waste as a function of decay time.



Fig. D.6. Thermal power of FFTF spent fuel, high-level waste, and structural material waste as a function of decay time.
Appendix D.3. Comparison of the Toxicity of Fuel Cycle Materials from a Single Reactor







Fig. D.8. Toxicity of LMFBR spent fuel, high-level waste, and structural material waste as a function of decay time.



Fig. D.9. Toxicity of FFTF spent fuel, high-level waste, and structural material waste as a function of decay time.

Appendix E. COMPARISON OF PWR, LMFBR, AND FFTF FUEL CYCLE MATERIAL CHARACTERISTICS FOR A SINGLE MATERIAL

Appendix E.l. Comparison of PWR, LMFBR, and FFTF Fuel Cycle Material Radioactivity for a Single Material

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Fig. E.l. Radioactivity of PWR, LMFBR, and FFTF spent fuel as a function of decay time.



Fig. E.2. Radioactivity of PWR, LMFBR, and FFTF high-level waste as a function of decay time.



Fig. E.3. Radioactivity of PWR, LMFBR, and FFTF structural material waste as a function of decay time.

Appendix E.2. Comparison of PWR, LMFBR, and FFTF Fuel Cycle Material Thermal Power for a Single Material



Fig. E.4. Thermal power of PWR, LMFBR, and FFTF spent fuel as a function of decay time.



Fig. E.5. Thermal power of PWR, LMFBR, and FFTF high-level waste as a function of decay time.



Fig. E.6. Thermal power of PWR, LMFBR, and FFTF structural material waste as a function of decay time.

Appendix E.3. Comparison of PWR, LMFBR, and FFTF Fuel Cycle Material Toxicity for a Single Material

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Fig. E.7. Toxicity of PWR, LMFBR, and FFTF spent fuel as a function of decay time.



Fig. E.8. Toxicity of PWR, LMFBR, and FFTF high-level waste as a function of decay time.

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Fig. E.9. Toxicity of PWR, LMFBR, and FFTF structural material waste as a function of decay time.

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